

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8707150068 DOC. DATE: 87/07/08 NOTARIZED: NO DOCKET #
 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power & Light Co 05000261
 AUTH. NAME AUTHOR AFFILIATION
 SAYRE, D. Carolina Power & Light Co.
 MORGAN, R. E. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-014-00: on 870611, during zero power physics testing, valve packing failure resulted in primary sys leakage of approx 23 gallons per minute. Valve packing replaced under special procedure & plant heatup reinitiated. W/870708 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD2-1 LA	1 1	PD2-1 PD	1 1
	ECCLESTON, K	1 1		
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	DEDRO	1 1
	NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/RAB	1 1
	NRR/DREP/RPB	2 2	NRR/PMAS/ILRB	1 1
	NRR/PMAS/PTSB	1 1	REG FILE 02	1 1
	RES DEPY GI	1 1	RES TELFORD, J	1 1
	RES/DE/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

NRC Form 366
(9-83)U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
H. B. Robinson Steam Electric Plant, Unit No. 2

DOCKET NUMBER (2)

0 5 0 0 0 2 6 1

PAGE (3)

1 OF 0 2

TITLE (4)
Loss of RHR

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)								
0	6	11	8	7	0	1	4	0	0	7	0	8	8	7	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)													
POWER LEVEL (10)	0	0	0	20.402(b)			20.406(c)			50.73(a)(2)(iv)			73.71(b)		
				20.406(a)(1)(i)			50.38(e)(1)			50.73(a)(2)(v)			73.71(c)		
				20.406(a)(1)(ii)			50.38(e)(2)			50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
				20.406(a)(1)(iii)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(A)					
				20.406(a)(1)(iv)			50.73(a)(2)(iii)			50.73(a)(2)(viii)(B)					
20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Don Sayre, Senior Specialist - Regulatory Compliance	AREA CODE 8 0 3 3 8 3 - 1 2 4 2

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B	P	I	S	V				N

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 11, 1987, during zero power physics testing following a refueling outage, a valve packing failure resulted in primary system leakage of approximately 23 gallons per minute.

An Unusual Event was declared.

Initial indications were that the leak was from the #1 seal of "C" Reactor Coolant Pump. Based on an erratic standpipe level and high temperature in the seal leakoff line, the pump was stopped. Later, similar indications were noted for the "B" Reactor Coolant Pump and it too was stopped.

Subsequent investigation of the leakage determined the source of the leak to be Residual Heat Removal valve RHR-750 inside containment, based on high temperature in its leakoff line. Common piping is shared by this line and the pump seal leakoff lines.

The packing was replaced, the Unusual Event terminated, and Plant startup reinitiated. The need for additional long-term corrective action is being evaluated.

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S PDR

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NRC Form 388A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31 85

FACILITY NAME (1) H. B. Robinson S. E. Plant, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 8 7 - 0 1 4 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 388A's) (17)

EVENT DESCRIPTION

On June 11, 1987, while performing zero power physics testing following a refueling outage, the packing on valve RHR-750 in containment failed. This resulted in a primary system leak of approximately 23 gallons per minute.

An Unusual Event was declared at 1730 hours that day, and the NRC was notified via the Emergency Notification System.

At the beginning of the event, Plant operators noted a sudden increase in unidentified RCS leakage. Initially, the leakage "appeared" to be from the #1 seal of "C" Reactor Coolant Pump (RCP) based on indications of an erratic level in the RCS standpipe and high temperature in the seal leakoff line. The pump was stopped. Reactor shutdown was initiated. Similar indications of high leakoff temperature and erratic standpipe level were then noted for "B" RCP and it was also stopped.

Technical Specification 3.1.5.2 requires the Reactor to be placed in the hot shutdown condition if the primary system leakage exceeds 10 gallons per minute.

Subsequent investigation of the leakage indicators in the Control Room determined high temperature in the leakoff line from RHR-750. The size of the leak and the common piping for the valve leakoff and pump seal leakoff lines apparently had created the original indication of the source of the leak as the RCP seal.

CAUSE

The packing failure on RHR-750 was the source of the primary system leak.

CORRECTIVE ACTION

The valve packing was replaced under a Special Procedure. The Unusual Event was terminated at 1000 hours on June 12, 1987, and Plant heatup reinitiated.

The need for additional long-term corrective action is being evaluated to preclude recurrence of the event.



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT
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JUL. 08 1987

Robinson File No: 13510C

Serial: RNP/87-3072

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 87-014

Dear Sir:

The enclosed Licensee Event Report (LER) is submitted in accordance with the Licensee Event Report System of 10 CFR 50.73. The format of the LER follows the recommendations of NUREG-1022, September 1983.

Very truly yours,

R. E. Morgan
General Manager
H. B. Robinson S. E. Plant

DAS:lko

Enclosure

cc: J. N. Grace
H. E. P. Krug
INPO

IE22
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