

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8706300222 DOC. DATE: 87/06/25 NOTARIZED: NO DOCKET #  
 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power & Light Co 05000261  
 AUTH. NAME AUTHOR AFFILIATION  
 ABBOTT, R. Carolina Power & Light Co.  
 MORGAN, R. E. Carolina Power & Light Co.  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-008-00: on 870527, reactor trip occurred. Caused by  
 miswiring of relay PC-457AX. Relay rewired. Maint Surveillance  
 Test MST-011 will be revised to require reactor trip  
 breakers to be in open position before test. W/870625 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3  
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

## NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD2-1 LA	1 1	PD2-1 PD	1 1
	ECCLESTON, K	1 1		
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	DEDRO	1 1
	NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSB	1 1	NRR/DEST/SCB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/RAB	1 1
	NRR/DREP/RPB	2 2	NRR/PMAS/ILRB	1 1
	NRR/PMAS/PTSB	1 1	REG FILE 02	1 1
	RES DEPY GI	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 42 ENCL 40

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. Robinson Steam Electric Plant Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 1				PAGE (3) 1 OF 0 2	
TITLE (4) Reactor Trip at Cold Shutdown															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0 5	2 7	8 7	8 7	0 0 8	0 0 0	0 6	2 5	8 7					0 5 0 0 0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)													
N		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)	
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)	
0 0 0		20.406(a)(1)(B)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)					
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME Roy Abbott, Senior Specialist - Regulatory Compliance										TELEPHONE NUMBER					
										AREA CODE 8 0 3		3 8 3 - 1 2 4 3			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs						
D															
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 27, 1987, during a refueling outage, RCS heatup was in progress with Tave at 180°F. At 0725 hours, the Plant experienced a reactor trip signal. The trip signal was received on "A" train while primary instrumentation channels were being simulated in preparation for performing Maintenance Surveillance Test, "Reactor Protection Logic at "0" Power", MST-011. An incorrectly wired relay in conjunction with a simulated pressurizer pressure signal satisfied the two out of three logic which initiated the trip.

The NRC was notified at 0940 hours in accordance with 10 CFR 50.72(b)(2)(ii).

The relay in question was replaced during the Refueling Outage when it was apparently miswired.

The relay was rewired and tested satisfactory. MST-011 is being revised to require the reactor trip breakers to be open prior to initiating the test to avoid an unnecessary trip.

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PDR ADOCK 05000261  
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  H. B. Robinson Unit 2	DOCKET NUMBER (2)  0 5 0 0 0 2 6 1 8 7 — 0 0 8 — 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 388A's) (17)

EVENT DESCRIPTION

On May 27, 1987, during a refueling outage, RCS heatup was in progress with Tave at 180°F. At 0725 hours, the Plant experienced a reactor trip signal. The trip signal was received on "A" train while primary instrumentation channels were being simulated in preparation for performing Maintenance Surveillance Test, "Reactor Protection Logic at "0" Power", MST-011. An incorrectly wired relay in conjunction with a simulated pressurizer pressure signal satisfied the two out of three logic which initiated the trip.

CAUSE

When the RCS is being heated to 200°F, the control rods are raised slightly off the bottom to prevent binding due to temperature changes, and the reactor trip breakers are in a closed position. Concurrent with Plant heatup, MST-011 was being performed as part of return to power operation. Since the MST is performed at cold shutdown, several parameters are simulated to establish normal operating conditions in the circuit. Relay PC-457AX, another pressurizer pressure bistable had been replaced during the refueling outage. MST-011 was also, to serve as post-maintenance testing for the relay replacement. However, this relay had been miswired during replacement. The miswiring of relay PC-457AX in conjunction with the simulated pressurizer pressure signal during heatup (RTBs closed) caused an erroneous trip signal.

CORRECTIVE ACTION

PC-457AX was rewired, and MST-011 was used as the acceptance test.

MST-011 will be revised to require the reactor trip breakers in the open position before performing the procedure.



Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT  
POST OFFICE BOX 790  
HARTSVILLE, SOUTH CAROLINA 29550  
JUN 25 1987

Robinson File No: 13510C

Serial: RNP/87-2824

United States Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
LICENSEE EVENT REPORT 87-008

Dear Sir:

The enclosed Licensee Event Report is submitted in accordance with the Licensee Event Report System of 10 CFR 50.73. The format of the LER follows the recommendations of NUREG-1022 of September 1983.

Very truly yours,

R. E. Morgan  
General Manager  
H. B. Robinson S. E. Plant

RCA:sdm

Enclosure

cc: J. N. Grace  
H. E. P. Krug  
INPO

IF22  
11