

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8102200583 DOC. DATE: 81/02/13 NOTARIZED: NO DOCKET #
 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261
 AUTH. NAME: STARKEY, R.B. AUTHOR AFFILIATION: Carolina Power & Light Co.
 RECIP. NAME: RECIPIENT AFFILIATION: Region 2, Atlanta, Office of the Director

SUBJECT: LER 81-004/03L-0: on 810116, leak discovered on path from
 pressurizer through pressurizer liquid space sample cooler
 to component cooling water sys. Caused by normal wear to
 sample cooler tube bundles. Bundle replaced.

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	A/D PLANT SYS10	1	1	A/D RAD PROT 11	1	1
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	AE0D	3	3	ASLBP/J.HARD	1	1
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	CONT SYS BR 17	1	1	CORE PERF BR 18	1	1
	DIR, DIV OF LIC	1	1	DIR, ENGINEER I20	1	1
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	EFF TR SYS BR23	1	1	EQUIP QUAL BR25	1	1
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	MECH ENG BR 33	1	1	MPA	3	3
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	NSIC 05	1	1	TERA: DOUG MAY	1	1

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7 8 60 61 63 69 74 75 80

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

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At 1228 hours during normal operation on 1-16-81, indication of leakage to the

CCW System prompted a RCS leak rate test which verified approximately 2.37 gpm

leakage at 1244 hours. Subsequent investigation revealed that the leak path was

from the pressurizer through the pressurizer liquid space sample cooler to the

CCW System via a leaking tube bundle in the sample cooler. The sample line isolation valves were closed and the leakage terminated at 1245. The leakage experienced is contrary to the requirements of Technical Specification 3.1.5.2 and is reportable per Technical Specification 6.9.2.b(2). Since the leakage was totally contained within a closed plant radioactive system, there was no threat to either plant or public health or safety.

7 8 9		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE			
0	9	P	B	E		F		H	T	E	X	C	H	C		Z	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21			
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.							
18	1	8	1	0	0	4	/	0	3	L	0						
21	22	23	24	25	26	27	28	29	30	31	32						
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
A	Z	Z	Z	0	0	0	0	Y		Y		A	S	1	3	5	
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

1	8	The root cause of the event was the sample cooler tube bundle leakage believed due
1	1	to normal operational wear. The immediate corrective action taken to eliminate the
1	2	leakage was the closure of the sample line isolation valves. The sample cooler tube
1	3	bundle was subsequently replaced to prevent recurrence. Additional evaluation of
1	4	the event will be conducted to determine if further corrective action is appropriate.

80

7 8 9
FACILITY STATUS (28) 1 5 E 1 0 0 (29) N/A (30) OTHER STATUS
% POWER
DISCOVERY DESCRIPTION (32) Operator Observation
METHOD OF DISCOVERY (31) A
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60
ACTIVITY CONTENT
RELEASED OF RELEASE (33) 1 6 Z 1 0 0 (34) N/A (35) AMOUNT OF ACTIVITY
LOCATION OF RELEASE (36) N/A
PERSONNEL EXPOSURES
NUMBER (37) 1 7 0 0 0 (38) Z (39) DESCRIPTION N/A
PERSONNEL INJURIES
NUMBER (40) 1 8 0 0 0 (41) DESCRIPTION N/A
LOSS OF OR DAMAGE TO FACILITY
TYPE (42) 1 9 Z (43) DESCRIPTION N/A
PUBLICITY
ISSUED (44) 2 0 N (45) DESCRIPTION N/A
NRC USE ONLY
78 69 80
8102200, 723
NAME OF PREPARER R. B. Starkey, Jr.
PHONE: (803) 383-4524

Supplemental Information

For

Licensee Event Report 81-004

1. Cause Description and Analysis

On 1-16-81 at 1228 hours, the combination of a volume control tank makeup signal, an increasing component cooling water (CCW) surge tank level and an increasing radiation reading from the CCW System radiation monitor alerted the operators to the possibility of a Primary Coolant System leak. A RCS leak test was initiated and, at 1244 hours, RCS leakage was verified and determined to be 2.37 gpm. At 1245 hours the pressurizer liquid sample line isolation valves, which are normally opened during day shift operations for sampling, were promptly closed. A second RCS leak test was performed and, at 1300 hours, the RCS leak rate was determined to be essentially zero. No further increase in CCW surge tank level or CCW radiation monitor readings occurred following the closure of the sample line isolation valves. The leak path was subsequently identified as from the pressurizer liquid space through the sample line into the sample cooler and into the CCW System via a leaking tube bundle in the cooler. The leakage experienced was in excess of the limits allowed by Technical Specification Section 3.1.5.2 and is reportable in accordance with Technical Specification 6.9.2.b(2).

The cause of the event was the sample cooler tube bundle leakage. The tubing leakage is attributed to normal wear.

Since the RCS leakage was totally contained within the CCW System, which is a closed radioactive system, there was no threat to either plant or public health or safety.

2. Corrective Action

The sample line isolation valves were closed to eliminate further leakage as immediate corrective action.

3. Corrective Action to Prevent Further Occurrence

The sample cooler tube bundle was removed and replaced. Although the leakage was probably a result of normal operational wear, further evaluation of the event will be carried out to determine if there are any generic implications which could require further action.