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SUBJECT: Provides status of util activities re seismic qualification
 of auxiliary feedwater sys per tecon concerning Generic Ltr
 81-14. Recirculation line analysis will be completed by
 Jul 1984.

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FEB 22 1984

Director of Nuclear Reactor Regulation
Attention: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
United States Nuclear Regulatory Commission
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
SEISMIC QUALIFICATION OF AUXILIARY
FEEDWATER SYSTEM (AFWS)

Dear Mr. Varga:

SUMMARY

The purpose of this letter is to document the status of Carolina Power & Light Company's (CP&L) activities concerning the seismic qualification of AFWS branch lines at H. B. Robinson Steam Electric Plant Unit No. 2 (HBR2). This is in response to concerns expressed by NRC staff in recent telephone conversations about Generic Letter 81-14.

DETAILS

The main suction and discharge piping of the AFWS is designed to Seismic Class I criteria. However, as shown on the attached diagram, there are several branch lines to be considered. These are:

1. "A" motor-driven AFW pump recirculation line
2. "B" motor-driven AFW pump recirculation line
3. Steam driven AFW pump recirculation line
4. Service water back-up suction source
5. Deep-well water back-up suction source
6. Various branch lines

The service water and deepwell suction sources are considered Seismic Class I up to and including locked closed valves AFW-24, SW-118, and DW-19, DW-21, respectively. These locked closed valves are considered to be equivalent to automatic isolation. The seismic qualification methodologies and acceptance criteria used for the AFWS piping are provided in Chapter 3 of the Final Safety Analysis Report (FSAR) and were summarized in our response to IE Bulletin 79-14 (Serial No. 81-559 dated March 31, 1981).

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ACTION

The pump recirculation lines are currently being reviewed. The present plans are to seismically analyze the motor driven and the steam driven AFW pump recirculation lines up to and including the existing isolation valve and flow orifice. In addition, analysis is currently being performed to verify that if a pipe breaks further downstream than the orifice, the flow loss through the orifice would be such that the minimum required AFW flow to the steam generators will be maintained.

In addition, CP&L will evaluate representative branch lines to provide reasonable assurance that branch line failure will not occur under seismic loading. If necessary, CP&L will modify the lines to ensure that the minimum required AFW flow will be maintained.

CONCLUSION

The recirculation line analysis will be completed by July 1984. If any modifications are determined to be necessary, CP&L will inform you of our plans and schedule at that point. The branch line evaluation will be completed by December 1984.

In summary, there is currently no evidence to indicate that the lines in question are not fully capable of meeting their original design function. If you have any questions on this subject, please contact a member of the Nuclear Licensing Staff.

Yours very truly,

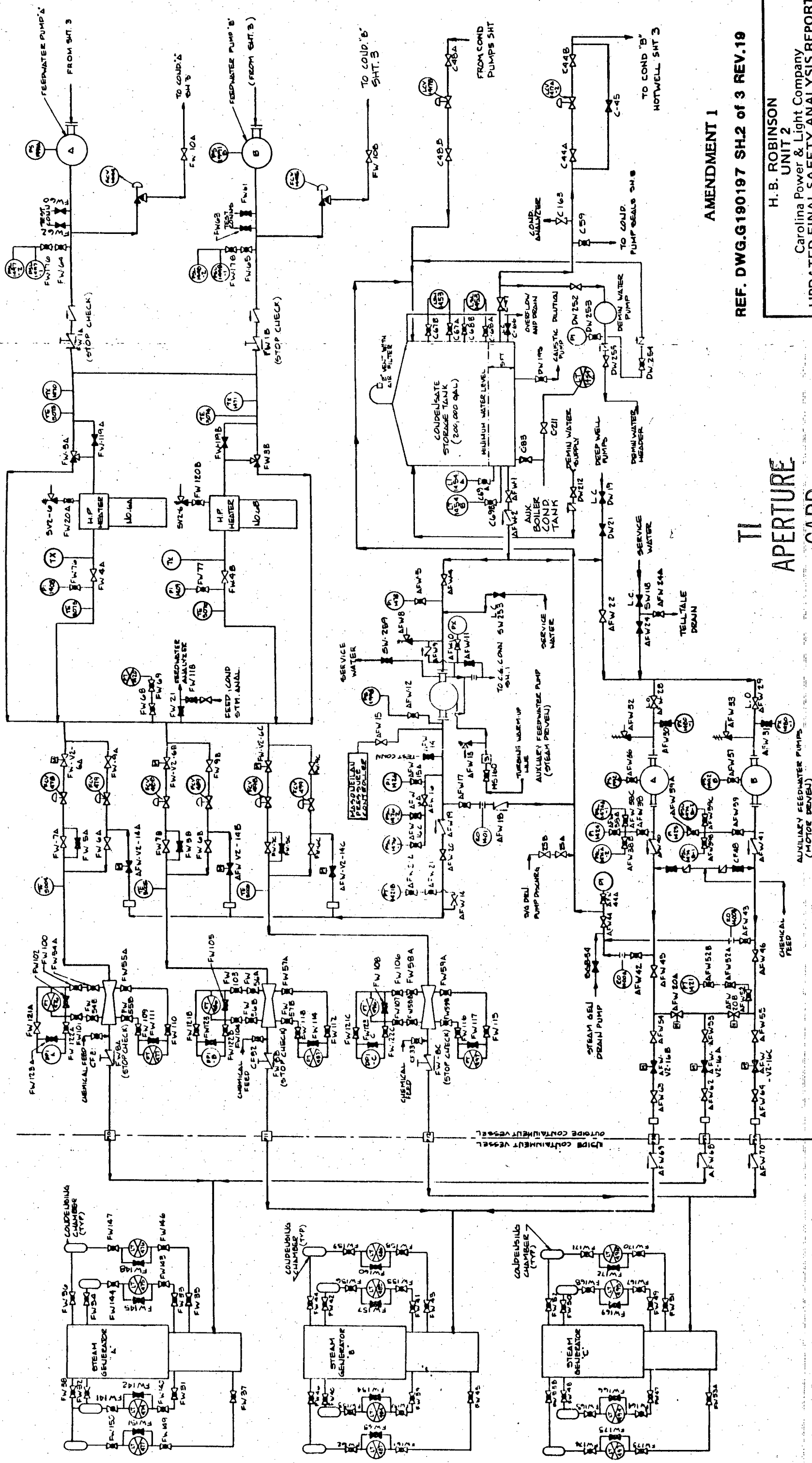


A. B. Cutter
Vice President

Nuclear Engineering & Licensing

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Attachment

cc: Mr. J. P. O'Reilly (NRC-RII)
Mr. G. Requa (NRC)
Mr. Steve Weise (NRC-HBR)



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AMENDMENT 1

REF. DWG.G190197 SH.2 of 3 REV.19

H. B. ROBINSON UNIT 2
Carolina Power & Light Company
UPDATED FINAL SAFETY ANALYSIS REPORT
FEEDWATER, CONDENSATE, AND AIR EVACUATION SYSTEM
SHEET 2 OF 3
FIGURE 10.1.0-5

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