

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261  
 AUTH. NAME AUTHOR AFFILIATION  
 CUTTER, A. B. Carolina Power & Light Co.  
 RECIP. NAME RECIPIENT AFFILIATION  
 VARGA, S. A. Operating Reactors Branch 1

SUBJECT: Informs that debris screens for containment vent & purge sys  
 will be installed during Jul 1984 steam generator  
 replacement outage. Until completion of investigation re need  
 for debris screens, venting will continue, per 811221 SER.

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Carolina Power & Light Company

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NOV 18 1983

Director of Nuclear Reactor Regulation  
Attention: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
United States Nuclear Regulatory Commission  
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
PURGING AND VENTING OF CONTAINMENT

Dear Mr. Varga:

Your letter to Carolina Power & Light Company (CP&L) dated December 21, 1981 required, among other items, debris screens to be provided for the containment purge/vent systems at H. B. Robinson Steam Electric Plant Unit No. 2 (HBR2) prior to permitting purging. CP&L's response, dated January 29, 1982 stated that we would investigate the need for debris screens; however, the description of the purge and vent valve system in the Safety Evaluation Report (SER) attached to your December 21, 1981 letter, seemed to indicate sufficient protection to insure that valve failure due to debris would not occur. In the interim, until completion of the investigation, CP&L believes that there is sufficient justification in the SER for continued venting.

The debris screen study has not yet been completed. However, as requested by you, and as discussed with you on October 3, 1983 by Mr. S. R. Zimmerman, CP&L will provide debris screens for the containment vent and purge systems. Preliminary design information is as follows. The debris screens will be located approximately one pipe diameter away from the inner side of the inboard isolation valves. The screens will be seismically analyzed in their installed position to prevent them from dislodging and damaging any other safety related equipment nearby. In addition, the screens will be designed to resist normal operating dynamic loads and LOCA static and dynamic loads. The debris screens will be constructed of steel mesh of the appropriate size. CP&L believes this design to be acceptable and will install the debris screens during the Steam Generator Replacement Outage which is currently scheduled to begin in July 1984.

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Mr. Steven A. Varga

- 2 -

NOV 18 1983

If no further questions are identified by your staff by three months prior to the Steam Generator Replacement Outage, CP&L will consider this design to be approved by your staff and the debris screen issue closed. Since the debris screens will now be installed, there is no need for CP&L to complete the investigation mentioned above.

If you have any further questions on this subject, please contact a member of the Nuclear Licensing Staff.

Yours very truly,



A. B. Cutter  
Vice President

Nuclear Engineering & Licensing

ONH/tda (80620NH)

cc: Mr. J. P. O'Reilly (NRC-RII)  
Mr. G. Requa (NRC)  
Mr. Steve Weise (NRC-HBR)