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 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261
 AUTH. NAME AUTHOR AFFILIATION
 EURY, L.W. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION
 EISENHUT, D.G. Division of Licensing

SUBJECT: Forwards response to Generic Ltr 83-10d re resolution of TMI
 Action Item II.K.3.5, "Automatic Trip of Reactor Coolant
 Pumps." Util participating in Westinghouse Owners Group to
 resolve issue.

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Carolina Power & Light Company

APR 22 1983

SERIAL: LAP-83-116

Mr. Darrell G. Eisenhut, Director
Division of Licensing
United States Nuclear Regulatory Commission
Washington, DC 20555


H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
RESOLUTION OF TMI ACTION ITEM II.K.3.5
"AUTOMATIC TRIP OF REACTOR COOLANT PUMPS" (GENERIC LETTER NO. 83-10d)

Dear Mr. Eisenhut:

Please find attached Carolina Power & Light Company's (CP&L) response to your letter of February 8, 1983 concerning TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps." Carolina Power & Light Company is a member of the Westinghouse Owners' Group (WOG) and as such will be participating in the WOG's program to resolve this issue. The attached response reflects that participation and is submitted in accordance with 10CFR50.54(f).

If you should have any questions on the attached response, please do not hesitate to contact our staff.

Yours very truly,

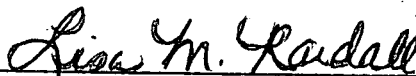

L. W. Eury
Senior Vice President
Power Supply

A046

JJS/kjr (6683JJS)
Attachment

cc: Mr. J. P. O'Reilly (NRC-RII)
Mr. G. Requa (NRC)
Mr. Steve Weise (NRC-HBR)

L. W. Eury, having been first duly sworn, did depose and say that the information contained herein is true and correct to his own personal knowledge or based upon information and belief.


Notary (Seal)

My commission expires: 5/18/83

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PDR ADDCK 05000261
P PDR

CAROLINA POWER & LIGHT COMPANY
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
PLAN FOR RESOLUTION OF TMI ACTION ITEM II.K.3.5
"AUTOMATIC TRIP OF REACTOR COOLANT PUMPS"

INTRODUCTION

The criteria for resolution of TMI Action Plan Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps" were stated in letters from Mr. Darrell G. Eisenhut of the Nuclear Regulatory Commission to all Applicants and Licensees with Westinghouse designed Nuclear Steam Supply Systems (83-10 c and d) dated February 8, 1983. The following represents the plan for demonstrating compliance with those criteria. In order to avoid confusion, the overall philosophy and plan will first be stated. Then, each section of the attachment to NRC letter 83-10 d will be addressed as to how the overall plan responds to each NRC criterion.

OVERALL PLAN

In the four years that have passed since the event at Three Mile Island, Westinghouse and the Westinghouse Owners' Group have held steadfastly to several positions relative to post accident reactor coolant pump (RCP) operation. First, there are small break LOCAs for which delayed RCP trip can result in higher fuel cladding temperatures and a greater extent of zircalloy-water reaction. Using the conservative evaluation model, analyses for these LOCAs result in a violation of the Emergency Core Cooling System (ECCS) Acceptance Criteria as stated in 10CFR50.46. The currently approved Westinghouse Evaluation Model for small break LOCAs was used to perform these analyses and found acceptable for use by the NRC in letters 83-10 c and d. Therefore, to be consistent with the conservative analyses performed, the RCPs should be tripped if indications of a small break LOCA exist.

Secondly, Westinghouse and the Westinghouse Owners' Group have always felt that the RCPs should remain operational for non-LOCA transients and accidents where their operation is beneficial to accident mitigation and recovery. This position was taken even though a design basis for the plant is a loss of off-site power. Plant safety is demonstrated in the Final Safety Analysis Reports for all plants for all transients and accidents using the most conservative assumption for reactor coolant pump operation.

In keeping with these two positions, a low RCS pressure (symptom based) RCP trip criterion was developed that provided an indication to the operator to trip the RCPs for small break LOCA but would not indicate a need to trip the RCP for the more likely non-LOCA transients and accidents where continued RCP operation is desirable. The basis for this criterion is included in the generic Emergency Response Guideline (ERG) Background Document (E-O Basic Revision, Appendix A). Relevant information regarding the expected results of using this RCP trip criterion can be derived from the transients which resulted from the stuck open steam dump valve at North Anna in 1979, the steam generator tube rupture at Prairie Island in 1980 and the steam generator

tube rupture at Ginna in 1982. The RCPs were tripped in all three cases. However, a study of the North Anna and Prairie Island transients indicated that RCP trip would not have been needed based on the application of the ERG trip criterion. The Ginna event, however, indicated a need to review the basis for the RCP trip criterion to allow continued RCP operation for a steam generator tube rupture for low head SI plants.

Thirdly, it has always been the position of Westinghouse and the Westinghouse Owners' Group that if there is a doubt as to what type of transient or accident is in progress, the RCPs should be tripped. Again, the plants are designed to mitigate the effects of all transients and accidents even without RCP operation while maintaining a large margin of safety to the public. The existing emergency operating procedures reflect this design approach.

Lastly, it remains the position of Westinghouse and the Westinghouse Owners' Group that RCP trip can be achieved safely and reliably by the operator when required. An adequate amount of time exists for operator action for the small break LOCAs of interest. The operators have been trained on the need for RCP trip and the emergency operating procedures give clear instructions on this matter. In fact, one of the initial operator activities is to check if indications exist that warrant RCP trip.

Westinghouse and the Westinghouse Owners' Group will undertake a two part program to address the requirements of NRC letters 83-10 c and d based on the aforementioned positions for the purpose of providing more uniform RCP trip criteria and methods of determining those criteria. In the first part of the program, revised RCP trip criteria will be developed which provides an indication to the operator to trip the RCPs for small break LOCAs requiring such action but will allow continued RCP operation for steam generator tube ruptures, less than or equal to a double-ended tube rupture. The revised RCP trip criteria will also be evaluated against other non-LOCA transients and accidents where continued RCP operation is desirable in order to demonstrate that a need to trip the RCPs will not be indicated to the operator for the more likely cases. Since this study is to be utilized for emergency response guideline development, better estimate assumptions will be applied in the consideration of the more likely scenarios. The first part of the program will be completed and incorporated into Revision 1 of the Emergency Response Guidelines developed by Westinghouse for the Westinghouse Owners' Group. The scheduled date for completion of Revision 1 is July 31, 1983.

The second part of the program is intended to provide the required justification for manual RCP trip. This part of the program must necessarily be done after the completion of the first part of the program. The schedule for completion of the second part of the program is the end of 1983.

The preferred and safest method of pump operation following a small break LOCA is to manually trip the RCPs before significant system voiding occurs.

No attempt will be made in this program to demonstrate the acceptability of continued RCP operation during a small break LOCA. Further, no request for an exemption of 10CFR50.46 will be made to allow continued RCP operation during a small break LOCA.

DETAILED RESPONSE TO NRC LETTER 83-10 d

Each of the requirements stated in the attachment to NRC letter 83-10 d will now be discussed indicating clearly how it will be addressed. The organization of this section of the report parallels the attachment to NRC letter 83-10 d.

I. Pump Operation Criteria Which Can Result in RCP Trip During Transients and Accidents.

1. Setpoints for RCP Trip

The Westinghouse Owners' Group (WOG) response to this section of requirements will be contained in Revision 1 to the Emergency Response Guidelines (ERGs) scheduled for July 31, 1983. For the H. B. Robinson Steam Electric Plant (HBR2), Revision 1 of the WOG ERGs will be implemented as Plant Specific Procedure prior to the end of the next refueling outage. Please see CP&L's April 15, 1983 response to NUREG-0737, Supplement 1 for additional details on our Revision 1 implementation schedule.

a) As stated above, Westinghouse and the Westinghouse Owners' Group are developing revised RCP trip criteria which will assure that the need to trip the RCPs will be indicated to the operator for LOCAs where RCP trip is considered necessary. The criteria will also ensure continued forced RCS flow for:

- 1) steam generator tube rupture (SGTR) (up to the design basis, double-ended tube rupture)
- 2) the other more likely non-LOCA transients where forced circulation is desirable (e.g., steam line breaks equal to or smaller than 1 stuck open PORV)

NOTE: Event diagnosis will not be used. The criteria developed will be symptom based.

The criteria being considered for RCP trip are:

- 1) RCS wide range pressure < constant
- 2) RCS subcooling < constant
- 3) Wide range RCS pressure < function of secondary pressure

Instrument uncertainties will be accounted for. Environmental uncertainty will be included if appropriate.

No partial or staggered RCP trip schemes will be considered. Such schemes are unnecessary and increase the requirements for training procedures and decision making by the operator during transients and accidents.

- b) The RCP trip criteria selected will be such that the operator will be instructed to trip the RCPs before voiding occurs at the RCP.
- c) The criteria developed in Item 1a above is not expected to lead to RCP trip for the more likely non-LOCA and SGTR transients. However, since continued RCP operation cannot be guaranteed, the emergency response guidelines provide guidance for the use of alternate methods for depressurization.
- d) The Emergency Response Guidelines contain specific guidance for detecting, managing and removing coolant voids that result from flashing. The symptoms of such a situation are described in these guidelines and in detail in the background document for the guidelines. Additionally, explicit guidance for operating the plant with a vaporous void in the reactor vessel head is provided in certain cases where such operation is needed. The existing HBR2 Emergency Operation Procedures (EOPs) include guidance on the detection and mitigation of voids in the reactor coolant system. The operators have also received training on this concern as a part of training courses such as "Mitigation of Core Damage." This guidance will be retained and revised to be consistent with Revision 1 of the WOG ERGs as applicable to HBR2. The operators will receive additional training as part of the implementation process for the revised EOPs.
- e) The current procedures at HBR2 require tripping the RCPs if essential services are lost. In order to restart the RCPs, current procedures require the verification that essential services are available.
- f) Discussed in 1a and 1c.

2. Guidance for Justification of Manual RCP Trip

The Westinghouse Owners' Group response to this section of requirements will be reported separately at the end of 1983. Carolina Power & Light Company will report upon the applicability of the WOG report to HBR2 three months after receipt of the report from the WOG.

- a) A significant number of analyses have been performed by Westinghouse for the Westinghouse Owners' Group using the currently approved Westinghouse Appendix K Evaluation Model for small break LOCA. This Evaluation Model uses the WFLASH Code. These analyses demonstrate for small break LOCAs of concern, if the RCPs are tripped 2 minutes following the onset of reactor conditions corresponding to the RCP trip setpoint, the predicted transient is nearly identical to those presented in the Safety Analysis Reports for all Westinghouse plants. Thus, the Safety Analysis Reports for all plants demonstrate compliance with requirement 2a. The analyses performed for the Westinghouse Owners' Group will be used to demonstrate the validity of this approach.

- b) Better estimate analyses will be performed for a limiting Westinghouse designed plant using the WFLASH computer code with better estimate assumptions. These analyses will be used to determine the minimum time available for operator action for a range of break sizes such that the ECCS acceptance criteria of 10CFR50.46 are not exceeded. It is expected that the minimum time available for manual RCP trip will exceed the guidance contained in N660. This will justify manual RCP trip for all plants.

3. Other Considerations

- a) The current RCP trip criteria (1300 psig) utilizes the RCS wide range pressure indication. This setpoint has been calculated per the current WOG generic guidelines and includes an accounting for normal and post-LOCA environment induced errors. The detailed sensing instrument information requested cannot be provided until after the WOG issues the revised RCP trip criteria. Therefore, CP&L will provide the requested information three months after the WOG formally issues the Revised RCP Trip Criteria.
- b) The Emergency Response Guidelines contain guidance for the timely restart of the reactor coolant pumps when conditions which will support safe pump start-up and operation are established. This guidance is presently contained in the HBR2 procedures for a MSLB and SGTR and will be in the procedures which incorporate the WOG ERGs.
- c) The HBR2 operators have been trained to trip the RCPs as soon as RCS pressure reaches 1300 psig during a LOCA, MSLB, or SGTR unless it is a planned and controlled depressurization. This training is reenforced during regularly scheduled simulator retraining. The operators will be trained on the revised RCP Trip Criteria and its priority during the implementation training for the new EOPs.

II. Pump Operation Criteria Which Will Not Result in RCP Trip During Transient and Accidents.

The preferred and safest method of operation following a small break LOCA is to manually trip the RCPs. Therefore, there is no need to address the criteria contained in this section.

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