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 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261
 AUTH. NAME: EURY, L.W. AUTHOR AFFILIATION: Carolina Power & Light Co.
 RECIP. NAME: VARGA, S.A. RECIPIENT AFFILIATION: Operating Reactors Branch 1

SUBJECT: Forwards "Pressurizer Safety & Relief Line Evaluation, Summary Rept," per NUREG-0737, Item II.D.1 request. Operability & structural integrity assured, contingent upon support adequacy. Temp measuring devices to be installed. *386 RPT*

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Carolina Power & Light Company

December 28, 1982

Office of Nuclear Reactor Regulation
ATTN: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
United States Nuclear Regulatory Commission
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
NUREG 0737 ITEM II.D.1.
PRESSURIZER SAFETY AND RELIEF VALVE
PIPING SYSTEM EVALUATION

Dear Mr. Varga:

As requested in NUREG-0737, Item II.D.1, Pressurizer Safety and Relief Valve Piping and Support Evaluation, Carolina Power and Light Company (CP&L) hereby submits the Pressurizer Safety and Relief Line Evaluation Summary Report for the H. B. Robinson Steam Electric Plant Unit 2 (HBR2). The report is enclosed for your review.

Contingent upon support adequacy, the operability and structural integrity of the system has been assured for all applicable loadings and load combinations. Support loads generated as a result of the thermal hydraulic analysis were greater than previously analyzed. The existing supports will be evaluated to determine what modifications are necessary. Any support modifications determined to be necessary will be accomplished during the next refueling outage which is scheduled to begin in February 1984.

Additionally, CP&L plans to install temperature measuring devices on the valve loop seals during the S/G Inspection Outage, or any outage of sufficient length prior to the S/G Inspection Outage. The data will be used to develop temperature profiles to verify the vendors assumptions regarding loop seal temperatures. However, an option to determine the HBR loop seal temperature profile analytically is being investigated. Any differences in the assumptions and actual data will be resolved within 90 days from the end of the S/G Inspection Outage.

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S. A. Varga

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December 28, 1982

If you have any questions regarding this report, please contact a member of my staff.

Yours very truly,



L. W. Eury
Senior Vice President
Power Supply

DCW/pgp (5887C8T2)
Attachment

cc: Mr. J. P. O'Reilly (NRC-RII)
Mr. G. Requa (NRC)
Mr. Steve Weise (NRC-HBR)