

# CATEGORY 1

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ACCESSION NBR: 9702110274      DOC. DATE: 97/02/09      NOTARIZED: NO      DOCKET #  
 FACIL: 50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C      05000261  
 AUTH. NAME      AUTHOR AFFILIATION  
 CHERNOFF, H.K.      Carolina Power & Light Co.  
 YOUNG, D.E.      Carolina Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 97-001-00: on 970110, TS violation occurred due to missed surveillance requirement. Caution cap will be remain in place until procedures are revised. W/970207 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1      SIZE: 4  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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AOE

**Carolina Power & Light Company**

Robinson Nuclear Plant  
3581 West Entrance Road  
Hartsville SC 29550

Robinson File No: 13510C

Serial: RNP-RA/97-0023

FEB 07 1997

United States Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261/LICENSE NO. DPR-23  
LICENSEE EVENT REPORT NO. 97-001-00

Gentlemen:

The enclosed Licensee Event Report (LER), is submitted in accordance with  
10 CFR 50.73(a)(2)(i)(B).

Very truly yours,

A handwritten signature in cursive script, appearing to read "Dale E. Young".

D. E. Young  
Plant General Manager

## Enclosure

c: Mr. L. A. Reyes, Regional Administrator, USNRC, Region II  
Ms. B. L. Mozafari, USNRC Project Manager, HBRSEP  
Mr. B. B. Desai, USNRC Senior Resident Inspector, HBRSEP

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PDR ADOCK 05000261  
S PDR

NRC FORM 366  
(4-95)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104  
EXPIRES 04/30/98**LICENSEE EVENT REPORT (LER)**(See reverse for required number of  
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION  
COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO  
THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING  
BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33)  
U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE  
PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET,  
WASHINGTON, DC 20503.

FACILITY NAME (1)

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NUMBER (2)

05000-261

PAGE (3)

1 OF 3

TITLE (4)

TECHNICAL SPECIFICATION VIOLATION DUE TO MISSED SURVEILLANCE REQUIREMENT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
01	10	97	97	-- 001	-- 00	02	09	97		05000
									FACILITY NAME	DOCKET NUMBER
										05000

  

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)				
POWER LEVEL (10)	100	20.2201(b)	20.2203(a)(2)(v)	<input checked="" type="checkbox"/>	50.73(a)(2)(i)	50.73(a)(2)(viii)
		20.2203(a)(1)	20.2203(a)(3)(i)		50.73(a)(2)(ii)	50.73(a)(2)(x)
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)		50.73(a)(2)(iii)	73.71
		20.2203(a)(2)(ii)	20.2203(a)(4)		50.73(a)(2)(iv)	OTHER
		20.2203(a)(2)(iii)	50.36(c)(1)		50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
		20.2203(a)(2)(iv)	50.36(c)(2)		50.73(a)(2)(vii)	

**LICENSEE CONTACT FOR THIS LER (12)**

NAME

H. K. Chernoff, Manager - Licensing/Regulatory Programs

TELEPHONE NUMBER (Include Area Code)

(803) 857-1437

**COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

**SUPPLEMENTAL REPORT EXPECTED (14)**EXPECTED  
SUBMISSION  
DATE (15)

MONTH DAY YEAR

YES

(If yes, complete EXPECTED SUBMISSION DATE).

X

NO

**ABSTRACT** (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On January 10, 1997, H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 was operating at 100% power. At 0149 hours Eastern Daylight Time, plant operators initiated a routine containment vessel pressure relief. At 0155 hours, operators discovered that a source check of the Radiation Monitoring System (RMS) radionoble gas monitor for the containment (i.e., monitor RMS-12) had not been performed as required by plant procedures and Technical Specifications (TS), and secured the pressure relief. Operators then performed the required source check of RMS-12, and the pressure relief was restarted at 0205 hours.

This event was caused by personnel error. The operator conducting the pressure relief failed to fully review procedural requirements established to perform the containment pressure relief prior to commencing this evolution. The operator responsible for this error was counseled. This event had no adverse impact on plant or public safety since the release was continuously monitored by plant vent monitors RMS-11, RMS-12, and RMS-14C. RMS-11 and RMS-12 would have isolated the release if an alarm had occurred.

Technical Specification (TS) Section 4.19, "Radioactive Effluent Instrumentation," requires, in part, that radioactive gaseous effluent monitoring instrumentation channels be demonstrated operable by performance of a source check at the frequencies shown in TS Table 4.19-2, "Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements." TS Table 4.19-2 requires that RMS-12 be source checked prior to each containment release. Since RMS-12 was not source checked prior to initiating the pressure relief, this condition constitutes a condition prohibited by TS. Accordingly, this report is submitted in accordance with 10 CFR 50.73(a)(2)(i)(B).

NRC FORM 366A

(4-95)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. DESCRIPTION OF EVENT

On January 10, 1997, H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 operating at 100 percent reactor power. At 0149 hours Eastern Daylight Time, plant operators initiated a routine containment vessel pressure relief. At 0155 hours, operators discovered that a source check of the Radiation Monitoring System (RMS) (EHS System Code:IL) radionoble gas monitor for the containment (i.e., monitor RMS-12) (EHS Component Code: MON) had not been performed as required by plant procedures and Technical Specifications (TS), and secured the pressure relief. Operators then performed the required source check of RMS-12, and the pressure relief was restarted at 0205 hours.

II. CAUSE OF EVENT

This event was caused by personnel error. The operator conducting the pressure relief failed to fully review procedural requirements and processes established to perform the containment pressure relief prior to commencing this evolution. In addition, the investigation of this event revealed that procedure guidance was not logically ordered to ensure that RMS-12 would be source checked prior to initiating a pressure relief.

III. ANALYSIS OF EVENT

Technical Specification (TS) Section 4.19, "Radioactive Effluent Instrumentation," requires that each radioactive gaseous effluent monitoring instrumentation channel be demonstrated operable by performance of the channel check, source check, channel calibration, and channel functional test operations at the frequencies shown in TS Table 4.19-2, "Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements." TS Table 4.19-2 requires that RMS-12 be source checked prior to each containment release. Since RMS-12 was not source checked prior to initiating the pressure relief, this condition constitutes a condition prohibited by TS. Accordingly, this report is submitted in accordance with 10 CFR 50.73(a)(2)(i)(B).

This event had no adverse impact on plant or public safety since the release was continuously monitored by plant vent monitors RMS-11, RMS-12, and RMS-14C. RMS-11 and RMS-12 would have isolated the release if an alarm had occurred.

NRC FORM 366A

(4-95)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)  
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

IV. CORRECTIVE ACTIONS

The operator that failed to source check monitor RMS-12 prior to initiating the pressure relief was counseled.

As an interim measure, a "caution cap" was placed on the Reactor Turbine Generator Board (RTGB) containment pressure control valve switch to caution operators of the need to verify the procedurally required data prior to initiating a pressure relief. This "caution cap" will remain in place until procedures are revised.

Procedure EMP-022, "Gaseous Waste Release Permits," and EMP-023, "Liquid Waste Release and Sampling," will be revised by March 21, 1997, so that the required information and actions are documented in a logical order. Operating Procedure (OP)-921, "Containment Air Handling," will be revised by March 21, 1997, to require the necessary data be recorded and source checks completed and verified prior to commencing a pressure relief.

V. ADDITIONAL INFORMATION

## A. Failed Component Identification

None

## B. Previous Similar Events

None