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SUBJECT: Repts significant change in application of acceptable ECCS
model that resulted in reduction of calculated peak fuel
cladding temperature by more than 50 F from temp
calculated for limiting transient using last model.

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10 CFR 50.46 (a)(3)(ii)

Carolina Power & Light Company

Robinson Nuclear Plant
3581 West Entrance Road
Hartsville SC 29550

Robinson File No.: 13510

Serial: RNP-RA/95-0120

JUN 28 1995

United States Nuclear Regulator Commission

ATTN: Document Control Desk

Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261/LICENSE NO. DPR-23

THIRTY-DAY REPORT PURSUANT TO 10 CFR 50.46 - REDUCTION IN
CALCULATED PEAK FUEL CLADDING TEMPERATURE

Gentlemen:

In accordance with 10 CFR 50.46 (a)(3)(ii) and 10 CFR 50.4, this letter reports a significant change in the application of an acceptable Emergency Core Cooling System model that resulted in a reduction of the calculated peak fuel cladding temperature by more than 50°F from the temperature calculated for the limiting transient (i.e., the Large Break Loss-of-Coolant Accident) using the last acceptable model. Based on this change being finalized on May 31, 1995, this report is required to be submitted by June 30, 1995.

The Core Operating Limits Report for H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 was revised in support of Cycle 17 and submitted to the NRC by letter dated June 1, 1995. The Cycle 17 analyses included a reanalysis of the Large Break Loss-of-Coolant Accident. The Cycle 17 analysis supports a hot channel factor ($F_{\Delta H}$) of 1.80 and a total core power peaking factor (F_{Q^T}) of 2.50. The calculated Peak Clad Temperature (PCT) is 2006°F. The limiting axial power shape is a center-peaked chopped cosine which is representative of beginning of cycle conditions. For comparison purposes, the previous result was a calculated PCT of 2134°F with $F_{Q^T} = 2.40$.

The reanalysis also accommodates minor changes in fuel design and possible changes in cycle operation. The new analysis was able to produce a lower calculated PCT with a higher peaking factor by refining the accumulator line loss coefficients in the Safety Injection model to reflect the results of previous plant accumulator dump tests.

Since this change resulted in a 50°F reduction in PCT and therefore the calculated PCT continues to comply with the criteria set forth in 10 CFR 50.46 (b), a proposed schedule for providing a reanalysis or taking other action as may be needed to show compliance with the requirements of 10 CFR 50.46 is not necessary.

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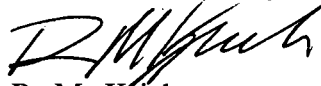
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Questions regarding this matter may be referred to Mr. A. L. Garrou at (803) 857-1544.

Very truly yours,



R. M. Krich

Manager - Regulatory Affairs

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