

April 24, 1979

FILE: NG 3513(R)

SERIAL: GD-79-1088

Mr. James P. O'Reilly, Director
United States Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30309

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261
LICENSE NO. DPR-23
RESPONSE TO IE BULLETIN 79-07

Dear Mr. O'Reilly:

IE Bulletin 79-07, dated April 14, 1979, requested information concerning the seismic stress analysis of safety-related piping. This letter provides Carolina Power & Light Company's response for the H. B. Robinson Plant. Seismic stress analysis for Robinson was performed by both Westinghouse and Ebasco, and the concern of the bulletin will be addressed separately.

Westinghouse provided a dynamic analysis of the reactor coolant loop using the WESDYN code in 1970. This analysis was performed using algebraic summation for intramodal responses, thus corresponding to Item 1a in Bulletin 79-07. The analysis has been redone using an updated version of WESDYN which used an absolute sum rather than algebraic summation. The maximum stress determined in the reanalysis was 3.0 ksi as compared to the allowable stress of 13.1 ksi. Documentation of the verification of the WESDYN code is contained in WCAP 8252, Revision 1, May 1977, "Documentation of Selected Westinghouse Structural Analysis Computer Codes." This report has been previously submitted to the NRC. The computer codes are proprietary to Westinghouse, and the subroutine of WESDYN requested by your bulletin will be provided by Westinghouse separately.

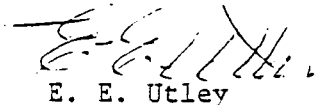
The remainder of the seismic Class I systems at H. B. Robinson were analyzed by Ebasco using a static analysis technique as discussed in the FSAR. This analytical technique is not among those identified in question No. 1 of Bulletin 79-07 and, therefore, Carolina Power & Light Company does not believe that the remainder of the questions apply to those systems analyzed by Ebasco.

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I trust this information is suitable for your use, and if there are any further questions, please contact our staff.

Yours very truly,



E. E. Utley
Senior Vice President
Power Supply

CSB/mf

cc: NRC Office of Inspection & Enforcement
Division of Reactor Operations Inspection
Washington, D. C. 20555