

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7903190281 DOC. DATE: 79/03/09 NOTARIZED: NO DOCKET #
 FACIL: 50-261 H. B. ROBINSON PLANT, UNIT 2, CAROLINA POWER AND LIGHT 05000261
 AUTH. NAME: STARKEY, R. B. AUTHOR AFFILIATION: CAROLINA POWER & LIGHT CO.
 RECIP. NAME: RECIPIENT AFFILIATION

SUBJECT: LER 79-003/03L-0 ON 790207:60 MIN CRITERIA OF CONSTANT AXIAL
 OFFSET CONTROL TARGET BAND LIMIT WAS EXCEEDING WHILE
 OPERATING AT POWER LEVEL GREATER THAN 50%. CAUSED BY
 TRANSIENT XENON CONDITIONS. POWER LEVEL DECREASED.

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 TITLE: INCIDENT REPORTS

NOTES: _____

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	16 EEB	1	1	17 AD FOR ENGR	1	1
	18 PLANT SYS BR	1	1	19 I&C SYS BR	1	1
	20 AD PLANT SYS	1	1	21 AD SYS/PROJ	1	1
	22 REAC SAFT BR	1	1	23 ENGR BR	1	1
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MAR 20 1979

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004

LICENSEE EVENT REPORT

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0	1	S	C	H	B	R	2	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5	
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE				30	57	CAT		58

CON'T.

REPORT SOURCE L 6 0 5 0 0 0 2 6 1 7 0 2 0 7 7 9 8 0 3 0 9 7 9 9

7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On February 7, 1979, the 60 minute criteria of the Constant Axial Offset Control

0 3 | (CAOC) target band limit was exceeded while operating at a power level greater than

0 4 | 50 percent. The power level was then decreased to below 50 percent as required by

0 5 | Technical Specification 3.10.2.8.a. This event is reportable in accordance with

0 6 | Technical Specification 6.9.2.b.2. On February 8, 1979, CAOC requirements were

0 7 | satisfied and the power level increased.

A number line is shown with tick marks at 7, 8, and 9. A box contains the digits '0' and '8'. The number 8 is marked on the line.

09		SYSTEM CODE Z Z		11	CAUSE CODE X		12	CAUSE SUBCODE Z		13	COMPONENT CODE Z Z Z Z Z Z						14	COMP. SUBCODE Z		15	VALVE SUBCODE Z		16
7	8	9	10		11		12		13					18		19		20					
17		LER/RO REPORT NUMBER		EVENT YEAR 7 9		21	22	SEQUENTIAL REPORT NO. 0 0 3		24	26	OCCURRENCE CODE 0 3		28	29	REPORT TYPE L		30	REVISION NO. 0		32		
ACTION AGEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		22	ATTACHMENT SUBMITTED		23	NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER		26			
X		H		B		Z		0 0 2		22	Y		23	N		Z		Z 9 9 9		26			
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The event resulted from transient xenon conditions following a reactor trip. The
1 1 | power level was limited to 50 percent until CAOC requirements were satisfied. Plant
1 2 | operations personnel will be instructed to limit power to below 50 percent when it
1 3 | appears that continued operation at higher power levels will result in exceeding the
1 4 | 60 minute restriction.

8 9 FACILITY STATUS (28) 30 OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)

1 5 C 0 8 3 NA A Operator observation

7 8 9 10 12 13 44 45 46 80

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 Z (33) NA (35)

7 8 9 10 11 44

AMOUNT OF ACTIVITY

LOCATION OF RELEASE (36)

45 80

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	NA	

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	8	0	0
0	0	0	40
		NA	

		LOSS OF OR DAMAGE TO FACILITY		
TYPE		DESCRIPTION		(43)
1	9	Z	(42) NA	

8 9 10
PUBLICITY
ISSUED N (44) DESCRIPTION (45) 7903190281 NRC USE ONLY
2 0
9 9 10 68 69 80

NAME OF PREPARER

R. B. Starkey, Jr.

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SUPPLEMENTARY INFORMATION FOR REPORTABLE
OCCURRENCE 79-03

I. Cause Description and Analysis:

At 1328 hours on February 6, 1979 a reactor trip occurred following sustained operation at 100 percent of rated power. At 0222 hours on February 7, 1979, while at 83 percent power (returning to full power) the total accumulated time that elapsed while operating outside of the CAOC target band exceeded the one hour criteria of Technical Specification 3.10.2.8.a. This required reactor power to be reduced and power operation restricted to 50 percent of rated power until 0511 hours on February 8, 1979 at which time the power level was increased to 100 percent. This constituted operation in a degraded mode as permitted by Technical Specification 3.10.2.8.a and is reportable in accordance with Technical Specification 6.9.2.b.2.

II. Corrective Action:

As required by Technical Specification 3.10.2.8.a, reactor power was reduced to no greater than 50 percent of rated power as indicated above.

III. Corrective Action To Prevent Further Occurrence:

Plant operations personnel will be instructed to limit the reactor power to 50 percent of rated power when it appears that continued operation at higher power levels will result in exceeding the 60 minute restriction.