NRC FORM 366 (7-77) Enclosure to Serial: RSEP/80-602

U. S. NUCLEAR REGULATORY COMMISSION

8005020279

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	S C H B R 2 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)
7 8	9 LICENSEE CODE 14	15 LICENSE NUMBER 25	26 LICENSE TYPE 30	57 CAT 58

CON'T 0 1	REPORT SOURCE L (6)	0 5 0 0 0 2 6 1 (7)	0 4 1 4 8 0 (8)	0 4 2 8 8 0 (9)
7 8	60	61 DOCKET NUMBER 68	69 EVENT DATE 74	75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On April 14, 1980, while Unit No. 2 was in the process of cooling down to inspect the

0 3 seals on "C" Reactor Coolant Pump, a primary to secondary leak of approximately .5 gpm

0 4 was detected in the "B" Steam Generator. This exceeded the maximum allowable leak

0 5 rate per steam generator (.35 gpm) permitted by Technical Specification 3.1.5.3. The

0 6 unit was brought to cold shutdown at 1245 on April 15 in accordance with Technical

0 7 Specification 3.1.5.3. This constitutes a reportable event in accordance with

0 8 Technical Specification 6.9.2.a.3.

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	C A (11)	E (12)	B (13)	H T E X C H (14)	F (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 0	—	0 0 9	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	Z (19)	C (20)	A (21)	0 3 3 6	Y (23)	Y (24)	N (25)	W 1 2 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Tube sheet inspections revealed four tubes with small but identifiable leaks. The

1 1 leaks are being repaired by explosive plugging. The cause for the tube failures is

1 2 being investigated and will be contained in a future report as required by Section

1 3 4.2.5.3.3 of the Technical Specifications.

1 4

1 5	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	G (28)	0 0 0 (29)	NA	A (31)	Operator Observation

1 6	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

1 7	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

1 8	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE	DESCRIPTION
	Z (42)	NA

PUBLICITY

2 0	ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)	NA	

NAME OF PREPARER R. B. Starkey, Jr. PHONE: (803) 383-4524

GPO 917-926

Enclosure to Serial:RSEP/80-602

SUPPLEMENTAL INFORMATION
FOR
LICENSEE EVENT REPORT 80-09

1. Cause Description and Analysis

At 1156 hours on April 14, 1980, while Unit No. 2 was in the process of shutting down to inspect the seals on "C" Reactor Coolant Pump, a sample test of steam generator water revealed a primary to secondary leak of an undetermined size. Further extensive testing and evaluation revealed by 2130 hours on April 14, 1980, that a primary to secondary leak of approximately .5 gpm existed in the "B" Steam Generator. This exceeded the maximum leakage per steam generator of .35 gpm allowed by Technical Specification Section 3.1.5.3. Cooldown operations, which had been suspended in order to verify the steam generator leak rate, were therefore immediately recommenced at 2130 hours on April 14, 1980. The unit was in cold shutdown at 1245 hours on April 15, 1980.

As a result of visual and eddy current inspections, as required by Section 4.2.5 of the Technical Specifications, four failed steam generator tubes were identified. The cause of the failures is currently being investigated and a follow-up report, as required by Section 4.2.5.3.3 of the Technical Specifications, will be submitted detailing the results of the investigation.

2. Corrective Action

The primary system was drained to below the generator channel head, and the secondary side was drained to the tube sheet. The tubes will be explosively plugged and will be verified as leak-tight by a leak test.

3. Corrective Action To Prevent Further Occurrence

The eddy current testing program required by Technical Specifications is being performed and defective tubes will be plugged in order to prevent the development of leaks. Further corrective action will be taken if investigations and evaluations indicate its necessity.