



Carolina Power & Light Company

December 10, 1979

*Central File*  
50-261

FILE: NG-3513 (R)

SERIAL: GD-79-3141

Mr. James P. O'Reilly  
U.S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, GA 30303

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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DRR-23  
RESPONSE TO IE BULLETIN 79-02, REVISION NO. 2

Dear Mr. O'Reilly:

In accordance with Revision 2 to IE Bulletin 79-02 dated November 8, 1979, a review of the revision requirements has been completed. Details and results of the verification, inspection, and testing program required by the Bulletin are maintained on site and are available for NRC inspection. The following information, in response to Revision 2 requirements, is provided for your review.

Maximum support loads used in the analytical work associated with Bulletin Action Items 1 and 2 are those associated with the original support design loads as identified by required reanalyses. All analytical work associated with Items 1 and 2 has been completed using these loads. A summary of the calculational methods and factors of safety used is provided in our July 9, 1979, submittal. Based on the additional guidance provided in Revision 2, a re-evaluation of the analyses performed using original support design loads has been initiated to ensure that the factor of safety requirements are met. If the re-evaluation identifies cases where the required factor of safety is not satisfied, appropriate corrective action will be taken. The re-evaluation is expected to be completed by February 15, 1979.

The inspections required by Item 4 of the Bulletin are addressed in the responses dated July 9, 1979, and July 20, 1979. As indicated by our previous responses, it is the intent of Carolina Power & Light Company to inspect and test all physically accessible expandable anchor bolts. An anchor bolt inspection and test program has been conducted and is presently in its final stages. All expandable anchor bolts have been inspected and tested or found to be inaccessible for testing. Those anchors listed as inaccessible due to location in areas inaccessible during plant operation will be inspected during

the next refueling outage. Those expandable anchors which are inaccessible due to physical limitations are currently being reexamined to determine if other methods of inspection or other test equipment may be used. Present schedule for completion of these inspections is December 14, 1979.

Item 4 of the Bulletin addresses the effect of preload on cyclic loading capacity. Preloading equal to or greater than the bolt design load was not verified or applied to the self-drilling type expansion anchor bolts inspected. The anchor bolt manufacturer does not recommend preloading this type of anchor. This recommendation is supported by dynamic tests performed by the manufacturer which demonstrate adequate performance of this type anchor without preload. Wedge type expansion anchors were verified by torquing to the values specified by the manufacturer.

Bulletin Item 5 addresses the use of expansion anchor bolts in concrete block walls to support Seismic Category 1 systems. Review and inspections have revealed no supports on Seismic Class I systems which use anchor bolts mounted in concrete block walls. Therefore, this item is not applicable to H. B. Robinson.

Bulletin Item 6 addresses use of expandable anchor bolts on supports without base plates. The inspection and test program described above and in previous responses included inspection of all pipe supports with expandable anchors. Therefore, this item has been resolved.

If you have any questions, please contact me or a member of my staff.

Yours very truly,



B. J. Furr  
Vice President  
Nuclear Operations

CSB/eaj\*

cc: Mr. N. C. Moseley