

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

REPORT

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

7 809

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

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NAME OF PREPARER

R. B. Starkey, Jr.

PHONE: (803) 383-4524

NRC USE ONLY

GPQ 917-926

SUPPLEMENTAL INFORMATION
FOR
LICENSEE EVENT REPORT 79-19

1. Cause Description and Analysis:

On June 25, 1979, as a result of review of information provided to Carolina Power and Light Company by Westinghouse Electric Corporation (NSSS Supplier), it was determined that the Plant Technical Specifications could have permitted reactor operation in a manner less conservative than assumed in the safety analyses. Specifically, recent analyses have revealed that a high energy break (feedwater line rupture) inside the reactor containment could result in a non-conservative bias to steam generator low level actuation of reactor protection due to ambient heating of the instrument's reference leg. This effect, it was determined, could cause as much as a 10% (of level span) error resulting in a delay of the Lo-Lo level reactor trip and auxiliary feedwater initiation. Technical Specifications currently include only a 5% margin to the safety analyses assumption for the level at which these actions occur. The Plant Operating Manual (POM), however, requires that this setpoint be at 15% of level span. Therefore, due to POM controls, no unsafe condition exists.

2. Corrective Action:

No immediate action was required due to POM controls of this setpoint and its current value of 15% of steam generator level span.

3. Corrective Action to Prevent Further Occurrence:

A Technical Specification change will be requested for the setpoint in paragraph 2.3.1.3(b) to be raised.