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10 CFR 50.59

U.S. Nuclear Regulatory Commission
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Southern Nuclear Operating Company
Vogtle Electric Generating Plant Units 3 and 4
Report of 10 CFR 50.59 Changes, Tests and Experiments
and 10 CFR 52 Appendix D Departure Report

Ladies and Gentlemen:

The U.S. Nuclear Regulatory Commission (NRC) issued the Vogtle Electric Generating Plant (VEGP) Units 3 and 4 combined licenses (COLs) (License Nos. NPF-91 and NPF-92, respectively) to Southern Nuclear Operating Company (SNC) on February 10, 2012.

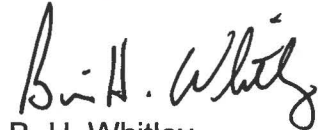
In accordance with 10 CFR 50.59(d)(2), VEGP Units 3 and 4 is required to submit a report to the NRC containing a brief description of any changes, tests or experiments made pursuant to 10 CFR 50.59(c), including a summary of the evaluation of each. This 10 CFR 50.59 report is for the period of December 8, 2012 to June 7, 2013. During that period there were no changes made, or tests or experiments conducted, pursuant to paragraph (c) of 10 CFR 50.59.

Additionally, in accordance with 10 CFR 52, Appendix D, paragraphs X.B.1 and X.B.3.b, VEGP Units 3 and 4 is required to submit a semi-annual report to the NRC containing a brief description of any plant-specific departures from the DCD, including a summary of the evaluation of each. This 10 CFR 52 Appendix D departure report, provided as Enclosure 1 to this letter, is for the period of December 8, 2012 to June 7, 2013.

If you have any questions regarding this letter, please contact Mr. Wes Sparkman at (205) 992-5061.

Respectfully submitted,

SOUTHERN NUCLEAR OPERATING COMPANY

A handwritten signature in black ink, appearing to read "B. H. Whitley". The signature is fluid and cursive, with the first letters of each word being capitalized and prominent.

B. H. Whitley

BHW/GAB/kms

Enclosure 1: Vogtle Electric Generating Plant (VEGP) Units 3 and 4, Semi-Annual Departure Report for the Period of December 8, 2012 to June 7, 2013

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Southern Nuclear Operating Company

ND-13-1216

Enclosure 1

Vogtle Electric Generating Plant (VEGP) Units 3 and 4

Semi-Annual Departure Report

for the Period of

December 8, 2012 to June 7, 2013

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Enclosure 1
VEGP Units 3 and 4 Semi-Annual Departure Report
for the Period of December 8, 2012 to June 7, 2013

LDCR / Departure Number: 2012-009

Title: Vents and Drains for Spent Fuel Pool Cooling System and Normal Residual Heat Removal System

Activity Description:

Three vent lines and two drain lines are added to the Spent Fuel Pool Cooling System (SFS) in Updated Final Safety Analysis Report (UFSAR) (plant-specific Design Control Document [DCD]) Figure 9.1-6 and a drain line is added to the Normal Residual Heat Removal System (RNS) in UFSAR Tables 3.2-3, 3.11-1 and 3I.6-3 and Figure 5.4-7. These vents and drains are added for maintenance draining and refilling of these systems. No system design functions are affected by the addition of these vents and drains.

Summary of Evaluation:

The addition of the vents and drains to the SFS and RNS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-022

Not a deferred change

Title: Investment Protection Short-Term Availability Controls (STAC) Upgrade and Conversion to Technical Requirements Manual (TRM)

Activity Description:

UFSAR (plant-specific DCD) Subsection 16.3, Investment Protection, is revised to reflect various upgrades in form and format, and to clarify the presentation of information. This activity relocates the Investment Protection Short-Term Availability Controls (STAC) from UFSAR Subsection 16.3 to a separate document (i.e., a Technical Requirements Manual [TRM]); however, the requirements relocated to the TRM will remain Tier 2 information. By virtue of being incorporated-by-reference into the UFSAR, and denoting the appropriate TRM content as Tier 2, the Tier 2 relationship is retained. In addition, changes were made to the STAC to provide consistency with Technical Specifications, conform to writers guide formatting, and to clarify requirements to improve usability.

Summary of Evaluation:

The relocation of the STAC from UFSAR Subsection 16.3 to the TRM, and the associated changes made to those requirements does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-037

Title: Changes to Passive Core Cooling System High Point Gas Accumulation Measurement

Activity Description:

This activity relocates the high point gas collection volume and measurement equipment in the Passive Core Cooling System (PXS) to the 107'-2" floor elevation, which is above the direct vessel injection (DVI) A/B piping and revises UFSAR (plant-specific DCD) Figure 6.3-2. With the gas collection point above this floor, the portion of piping above the floor is isolated from squib valve loads. Also, the gas collection volume in this revised configuration is less, to reduce the potential for a larger volume of gas creating voids in the DVI lines on gas expansion and blowback during a small break Loss of Coolant Accident (LOCA) event.

The design change to relocate the PXS high point gas collection volume and measurement is consistent with the gas accumulation assessment and mitigation of gas instrumentation system design features described in UFSAR (plant-specific DCD) Subsection 6.3.6.3.

Summary of Evaluation:

The relocation and reconfiguration of the high point gas collection volume and measurement equipment in the PXS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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VEGP Units 3 and 4 Semi-Annual Departure Report
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LDCR / Departure Number: 2012-038

Title: Add Relief Valves to UFSAR Figure 6.4-2

Activity Description:

This activity adds four relief valves to UFSAR (plant-specific DCD) Figure 6.4-2, Simplified Main Control Room Habitability System (VES) Piping and Instrumentation Diagram. These relief valves protect the VES Emergency Air Supply tanks from over pressurization. The valves are added to Figure 6.4-2 for consistency with the existing AP1000 design.

Summary of Evaluation:

Adding the relief valves to UFSAR Figure 6.4.2 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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VEGP Units 3 and 4 Semi-Annual Departure Report
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LDCR / Departure Number: 2012-050

Title: Changes In General Arrangement Drawings Due To Steam Generator Grating Changes

Activity Description:

UFSAR (plant-specific DCD) Figure 1.2-11 is revised to show the elevation of the grating around the steam generator upper manway area as 166'-1½" (revised from 166'-3¾"). The grating elevation around the steam generator upper manway area was lowered for personnel safety and constructability.

Summary of Evaluation:

Revising the elevation of the grating around the steam generator upper manway area does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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VEGP Units 3 and 4 Semi-Annual Departure Report
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LDCR / Departure Number: 2012-051

Title: Changes to Service Water System (SWS) P&ID

Activity Description:

UFSAR (plant-specific DCD) Figure 9.2.1-1 is revised to show piping class designations for portions of the SWS strainer backwash, normal and alternate blowdown lines, and normal makeup lines. In addition, the Circulating Water System (CWS) strainer backwash line is restored and the SWS position switches associated with the cooling tower bypass valves are removed.

These changes provide the proper interfaces between the SWS, Waste Water System (WWS), and CWS consistent with their intended functions.

Summary of Evaluation:

Revising the system piping class designations, removal of the SWS position switches and restoring the CWS strainer backwash line connection does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-053

Title: Change of Sensor Type for the Passive Core Cooling System Containment Flood-Up Level Instruments

Activity Description:

This activity revises the Passive Core Cooling System (PXS) containment (flood-up) water level monitoring instruments from thermocouples to magnetically actuated float level sensors as described in UFSAR (plant-specific DCD) Chapter 19, Appendix 19D. The magnetically actuated float type level sensors were determined to be superior to heated junction thermocouple devices based on a comparison of their complexity, measurement configuration and effectiveness, reliability, and operating experience.

The existing design functions of monitoring and indication in the Main Control Room for post-accident containment flooding, severe-accident containment flood-up monitoring, Post-Accident Monitoring requirements and back-up monitoring for containment sump leak detection are retained by this change.

Summary of Evaluation:

The PXS change to provide a magnetically actuated float type level sensor to monitor containment water level does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2012-055

Title: Changes to Structures and Layout of the Turbine Building

Activity Description:

Turbine Building structures and layout information is revised in the VEGP 3&4 UFSAR and in Tier 1 (plant-specific DCD). Changes are made to the door location on the motor-driven fire pump room, clarifying the column line designations for the southwest and southeast walls of the Turbine Building first bay, changing the floor to ceiling heights at three different elevations in the Turbine Building main area, increasing the top-of-wall elevation of the Turbine Building first bay and increasing the thickness of the first bay east and west walls.

A second portion of this departure includes increasing the Condenser Air Removal Stack Release elevation, updating fire protection summary information, increasing the height of the Heater Bay roof, adding a new upper Heater Bay elevation at 228'-9", and increasing the height of the Turbine Bay crane rail.

Summary of Evaluation:

Portions of this departure affected Tier 1 information in the AP1000 Certified DCD and Combined License (COL) Appendix C, and Tier 2* and Tier 2 in the UFSAR; therefore, a License Amendment and Exemption Request (LAR-12-006) was submitted to the NRC.

The NRC approved this departure and exemption on May 16, 2013 (ML13115A570) and issued License Amendment 007 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13115A858.

The second portion of this departure does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

That portion of the departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2012-056

Title: Normal Residual Heat Removal System (RNS) Heat Exchanger Drains

Activity Description:

This activity adds drain valves to the two RNS heat exchangers. Each RNS heat exchanger has five zones requiring draining and therefore requires five drain valve assemblies for complete draining during maintenance. The RNS design function is unchanged by the addition of the drain valves on the RNS heat exchangers.

Summary of Evaluation:

The addition of drain valves on the RNS heat exchangers does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2012-057

Title: Updating Equipment Qualification Information in the UFSAR

Activity Description:

This activity updates environmental qualification information in UFSAR (plant-specific DCD) Appendix 3D, Methodology for Qualifying AP1000 Safety-Related Electrical and Mechanical Equipment, to incorporate the latest environmental information consistent with a revised Master Equipment Qualification Environmental Summary Design Specification and add post-accident radiation dose information to be used for equipment qualification (EQ) of safety-related equipment within the Auxiliary Building, in accordance with Regulatory Guide (RG) 1.183.

Summary of Evaluation:

Updating and adding EQ information to the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2012-058

Title: Tier 1 Definition of Wall Thicknesses

Not a deferred change

Activity Description:

Revise VEGP 3&4 (plant-specific) Tier 1 information in Table 3.3-1, Definition of Wall Thicknesses for Nuclear Island Buildings, Turbine Building, and Annex Building to be consistent with UFSAR (plant-specific DCD) Tier 2 information and to provide clarity to support ITAAC closure or provide completeness of information.

Summary of Evaluation:

This activity affected Tier 1 information in the AP1000 Certified DCD and COL Appendix C, therefore a License Amendment and Exemption Request (LAR-12-008) was submitted to the NRC.

The NRC approved this departure and exemption on May 8, 2013 (ML13074A149) and issued License Amendment 006 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13074A178.

LDCR / Departure Number: 2012-061

Not a deferred change

Title: Simplification of Figure 9.1-6

Activity Description:

This activity changes the Spent Fuel Pool Cooling System Piping and Instrumentation Diagram, UFSAR (plant-specific DCD) Figure 9.1-6, Sheets 1 and 2, by adding vents and drains used for containment leak rate testing and by removing excess design details. The simplified figures together with associated FSAR text continue to provide sufficient understanding of the design bases, safety analyses and facility operation. The design basis system piping and instrumentation diagrams are not altered by this activity.

Summary of Evaluation:

Adding leak rate testing valves and removing excess detail from the Spent Fuel Pool Cooling System Piping and Instrumentation Diagram in the UFSAR does not impact the design function of any structure, system, or component (SSC) and does not result in a modification, addition to, or removal of a SSC such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-065

Title: Basemat Headed Reinforcement Spacing

Not a deferred change

Activity Description:

UFSAR (plant specific DCD) Figure 3.8.5-3, Sheet 7 is revised to show the #5 shear reinforcement bar spacing of 6 x 6 inches (revised from 6 x 12 inches) for the concrete under the sump on the radiologically controlled area of the auxiliary building. The nuclear island basemat shear reinforcement design continues to meet American Concrete Institute (ACI) 349 Code requirements and will not have an adverse impact on the strength of the nuclear island structures or the response of the structure to internal and external loads, including seismic loads.

Summary of Evaluation:

Increasing the shear reinforcement under the auxiliary building sump by decreasing the spacing between the bars does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or on the control of the reactions in the core design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2012-066

Title: Structural Steel Components

Activity Description:

The polar crane top of rail elevation shown in UFSAR (plant-specific DCD) Figure 3.8.2-1, Sheet 3 is revised from 228'-6 1/4" to 228'-5", along with other associated dimensions on that figure. The change in rail elevation is a result of a change in the profile and size of the rail selected in the final design. In addition, UFSAR Table 3.8.4-6 was revised to add American Society for Testing and Materials (ASTM) A490 structural bolting material and correct the description of ASTM A194. Also, a clarification was made in Subsection 3.8.4.6 regarding the use of industry specifications to clarify that the editions of industry specifications are those that are applicable after the start of construction or procurement activities.

Summary of Evaluation:

The change to the polar crane rail elevation and the change to Table 3.8.4-6 and Subsection 3.8.4.6 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-067

Title: PXS Screen Flow Limit Changes for RNS Injection to Prevent ADS Stage 4

Activity Description:

This activity changes UFSAR (plant-specific DCD) Subsection 6.3.2.2.7.1 to increase the maximum post-accident flow limit through the containment recirculation screens, the IRWST screens and RNS during post-LOCA injection or recirculation to 2600 gpm. The change is required to provide a flow range of 438 gpm (2600 gpm – 2162 gpm), which is large enough to account for RNS flow instrument error and flow control variations.

Summary of Evaluation:

Increasing the maximum flow through the containment recirculation screens, the IRWST screens and RNS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

LDCR / Departure Number: 2012-070

Title: Spent Fuel Pool Cooling System

Activity Description:

This activity changes the Spent Fuel Pool Cooling System Piping and Instrumentation Diagram, Figure 9.1-6, in the UFSAR (plant-specific DCD) to remove the Integrated Head Package (IHP) storage tank and associated piping and rerouted the drain line from the head stand to the Liquid Radwaste System (WLS) rather than the In-containment Refueling Water Storage Tank (IRWST). In addition, this activity adds level indication to the Cask Loading Pit.

The IHP design does not require shielding by water. This allows for the IHP storage tank and associated piping to be removed. The drain line from the head stand is rerouted to the WLS rather than the IRWST to support in-service inspections and prevent contaminants from entering the IRWST. In addition, an accurate method of measuring and verifying water level in the Cask Loading Pit is required to ensure water is available as a fuel storage pool makeup water source.

Summary of Evaluation:

This departure to the Spent Fuel Pool Cooling System does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2013-002

Not a deferred change

Title: Basemat Shear Reinforcement Design Spacing Requirements

Activity Description:

Revise Tier 2* information in UFSAR (plant-specific DCD) Subsection 3.8.5.5 to remove the direct reference to ACI 349 Subsection 11.8.3 and replace it with supplemental provisions based on criteria from ACI 349-01.

Summary of Evaluation:

This activity affected Tier 2* information in the UFSAR (plant-specific DCD), therefore a License Amendment Request (LAR-13-003) was submitted to the NRC.

The NRC approved this departure on February 26, 2013 (ML13051A780) and issued License Amendment 004 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13051A857.

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LDCR / Departure Number: 2013-003

Title: Basemat Shear Reinforcement Design Details

Not a deferred change

Activity Description:

Revise Tier 2* information and associated Tier 2 information in UFSAR (plant-specific DCD) Section 3.8, to change the licensing basis for development of basemat shear reinforcement from ACI 349 Appendix B to ACI 318-11, Section 12.6. The use of ACI 318 criteria for headed reinforcement results in longer shear ties and thicker concrete in areas below the elevator pits and a sump in the nuclear island basemat. The thicker concrete is accomplished by raising the floor of the elevator pits and sump in the nuclear island basemat resulting in a minor reduction in volume of the sump. The requirements for concrete cover over the reinforcement bars are also changed.

Summary of Evaluation:

This activity affected Tier 2* information and associated Tier 2 information in the UFSAR (plant-specific DCD), therefore a License Amendment Request (LAR-13-004) was submitted to the NRC.

The NRC approved this departure on March 1, 2013 (ML13056A446) and issued License Amendment 005 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13056A471.

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LDCR / Departure Number: 2013-006

Title: Turbine Building Eccentric and Concentric Bracing

Activity Description:

The VEGP 3&4 Tier 1 and the UFSAR (plant-specific DCD) is revised to change the bracing design in the non-seismic Turbine Building main area to use a system comprised of a combination of eccentrically braced frames and special concentrically braced frames, and the structural design code is changed to the International Building Code.

In addition, the design code for portions of the Annex Building is changed to the International Building Code.

Summary of Evaluation:

A portion of this departure affected Tier 1 information in the AP1000 Certified DCD and COL Appendix C and associated Tier 2 information in the UFSAR, therefore a License Amendment and Exemption Request (LAR-13-005) was submitted to the NRC.

The NRC approved this departure and exemption on May 21, 2013 (ML13121A365) and issued License Amendment 008 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13121A421.

The Annex Building portion of this departure does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

The Annex Building portion of this departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2013-012

Title: Service Water System (SWS) Changes

Activity Description:

This activity changes information on the Service Water System (SWS) in UFSAR (plant-specific DCD) Section 1.8, Section 9.2 and Figure 9.2.1-1. Changes include rerouting the SWS backflush flow line to the Turbine Building Sump, adding vibration and gearbox oil level indication to the SWS cooling tower fans, and changing a system interface boundary flag from the compressed air system to the SWS cooling tower basin bubbler level indication.

Summary of Evaluation:

This Service Water System departure does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2013-016

Not a deferred change

Title: Nuclear Island Walls Reinforcement Criteria

Activity Description:

UFSAR (plant-specific DCD) Section 3.8 and Appendix 3H is revised to provide alternative requirements for development of headed reinforcement in the licensing basis from ACI 349 Appendix B to include ACI 318-11, Section 12.6. This change clarifies the design and licensing basis for the headed reinforcement bars in locations including auxiliary building walls and walls within containment. The associated UFSAR (plant-specific DCD) figures for the auxiliary building wall reinforcement are also revised. The size and spacing of the shear reinforcement in these walls is replaced with a reference to the provided shear reinforcement. The use of shear ties with alternating 90 degree and 135 degree hooks to provide shear reinforcement in the walls in lieu of the headed reinforcement is also included as an alternative for the exterior walls below grade.

Summary of Evaluation:

This activity affected Tier 2* and associated Tier 2 information in the UFSAR, therefore a License Amendment Request (LAR-13-009) was submitted to the NRC.

The NRC approved this departure on May 22, 2013 (ML13122A102) and issued License Amendment 009 to Combined Licenses NPF-91 and NPF-92 for VEGP Units 3 and 4, respectively. The NRC Safety Evaluation is documented in ML13122A150.

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LDCR / Departure Number: 2013-018

Not a deferred change

Title: UFSAR Changes Associated with Nuclear Development Quality Assurance Manual (NDQAM) Version 11.0 and Other Organizational Changes

Activity Description:

This activity changes Initial Test Program (ITP) information in UFSAR (plant-specific DCD) Section 14.2, to clarify the scope, responsibilities and qualification of personnel for the ITP organization.

Summary of Evaluation:

The change to the ITP description in UFSAR Chapter 14 enhances and clarifies reporting relationships, responsibilities and qualifications of ITP personnel, and the scope of different phases of testing within the ITP, and does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2013-020

Title: WLS Monitor Tank Locations

Not a deferred change

Activity Description:

UFSAR (plant-specific DCD) Appendix 1A and Subsection 3.4.1.2.2.2 is revised to show the location of Liquid Radioactive Waste System (WLS) Waste Monitor Tanks at elevation 92'-6" and elevation 107'-2".

Summary of Evaluation:

Changing the location of the Waste Monitor Tanks in the Auxiliary Building does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

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LDCR / Departure Number: 2013-031

Not a deferred change

Title: Containment Vessel Coatings Conformance to Regulatory Guide 1.54, Revision 2

Activity Description:

This activity changes the licensing basis for protective coatings for the containment vessel shell and attachments to the containment vessel shell to reflect conformance to Revision 2 of Regulatory Guide (RG) 1.54, Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants. A detailed reconciliation was performed between Revision 2 and Revision 1 of RG 1.54, including the ASTM standards referenced within these RGs. The reconciliation demonstrated that the standards referenced within Revision 2 meet or exceed the standards referenced within Revision 1 of RG 1.54. The programmatic and operational aspects of these protective coatings will continue to conform to Revision 1 of RG 1.54.

Summary of Evaluation:

Revising the protective coatings for the containment vessel shell and attachments, to conform to RG 1.54, Revision 2, does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

This departure did not involve a change to Tier 1 information, Tier 2* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.