

Cracking Behavior of Irradiated Cast Austenitic Stainless Steels in Low-Corrosion-Potential Environments

Y. Chen,¹ B. Alexandreanu,¹ K. Natesan,¹ and A.S. Rao²

¹ Nuclear Engineering Division, Argonne National Lab, Argonne, IL 60439

² U.S. Nuclear Regulatory Commission, Rockville, MD 20852

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Cast austenitic stainless steels (CASS)

- **CASS are used in the cooling system of light water reactors for components with complex shapes**
 - pump casings, valve bodies, coolant piping, control rod guide tube spacers and housing etc.
- **The most widely used CASS steels are corrosion-resistant CF grades**
 - Typically CF grade CASS (CF-3, CF-8 and CF-3M ,CF-8M) whose compositions are similar to those of 300-series austenitic SSs.
 - CF-3 and CF-8 contain 19% Cr and 9% Ni
 - CF-3M and CF-8M are Molybdenum- containing steels
 - Because of high Cr content CF Grade steels possess
 - Corrosion and Oxidation Resistance and Mechanical properties
 - The strength and ductility is comparable to Wrought SS

Cast austenitic stainless steels (CASS)

- **Microstructure of CASS**

- (δ)-Ferrite and (γ)-Austenite Duplex microstructure
- Duplex Microstructure → provides Mechanical Strength of CASS
 - Tensile strength and Yield strength increases with (δ)-Ferrite up to 40% both at Room Temperature and elevated Temperature
- (δ)-Ferrite → Prevent the tendency for Hot Cracking → Welding
 - Improves the resistance to Sensitization and SCC

- **The Presence of δ - Ferrite**

- Detrimental Effect on Fracture Resistance when exposed to 300-500°C.
- It is because → δ - ferrite is vulnerable to Low-Temperature Thermal Aging
 - As a result both tensile strength and ductility are reduced

- **Mechanism for Loss of FT due to Thermal Aging in CASS**

- Nucleation of Cr-rich α' phase (by spinodal decomposition) and growth of α' - phase,
- precipitation of nickel- and silicon-rich G phase, γ - (austenite), and $M_{23}C_6$ carbides
- Large $M_{23}C_6$ carbides and/or Cr_2N nitrides precipitate at the ferrite/austenite phase bound

Note: Thermal aging: It is process that involves heating a material at a constant temperature for long period (in this case heating to 400°C and keep at that temperature for ~ 10,000 hrs.)

- The thermal aging embrittlement of CASS has been studied extensively
- Limited data exist in the open literature for thermal aging and neutron-irradiated CASS
- No data exists on the interaction between both thermal aging and neutron irradiation in CASS materials
 - Due to both cost and the experimental difficulties associated with the measurement
- The effect of neutron irradiation
 - Formation and growth of lattice defects leads to higher strength, lower ductility and toughness
 - Degradation of δ - ferrite much faster than for austenite
 - *fluence of $\sim 10^{17}$ n/cm² \rightarrow δ - ferrite and 10^{20} n/cm² \rightarrow γ - austenite (NRC- NUREG 7027)*
 - *fluence of 10^{17} - 10^{19} n/cm² \rightarrow δ - ferrite and 2×10^{20} – 6.7×10^{20} n/cm² \rightarrow γ - austenite (Industry - MRP-227 Meeting at USNRC Nov. 19th 2013)*
 - *Note: 10^{17} n/cm² \sim 0.00015 dpa and 2×10^{20} – 6.7×10^{20} n/cm² \sim 0.3 – 1 dpa*

Objective

- **The objective of this investigation is**
 - 1. to assess the extent of embrittlement of CASS from**
 - neutron irradiation (dose 0.08 dpa)
 - thermal aging (400°C, ~ 10,000 hrs)
 - neutron irradiation and thermal aging
 - 2. to evaluate their stress corrosion cracking (SCC) performance in low electrochemical potential (ECP) environments**
 - simulated PWR water with 2ppm Lithium and 1000 ppm Boron
 - high-purity water with dissolved oxygen (DO) < 10 ppb; conductivity ~ 20 μ S/cm



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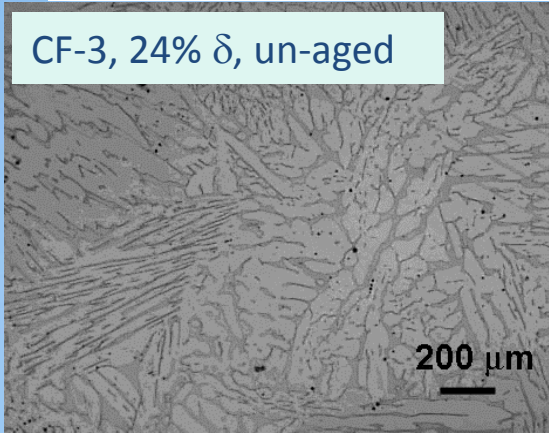
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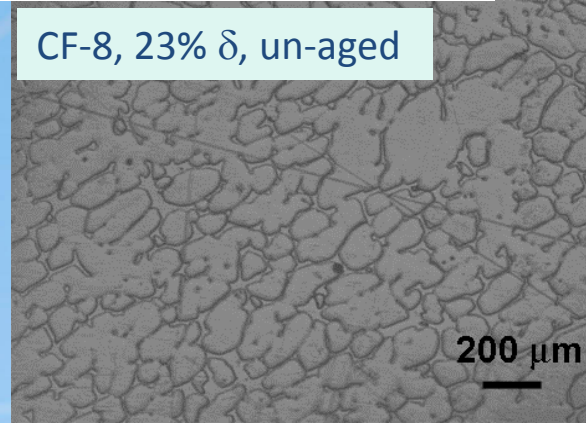
Materials

Cast Steel Grade	Ferrite Content	Heat ID.	Spec. ID	Thermal Aging Condition	Elemental Composition (wt. %)								
					Mn	Si	P	S	Mo	Cr	Ni	N	C
CF-3	24%	69	A	Unaged	0.63	1.13	0.015	0.005	0.34	20.18	8.59	0.028	0.023
			B	10,000 hrs @ 400°C									
CF-8	23%	68	C	Unaged	0.64	1.07	0.021	0.014	0.31	20.46	8.08	0.062	0.063
			D	10,000 hrs @ 400°C									

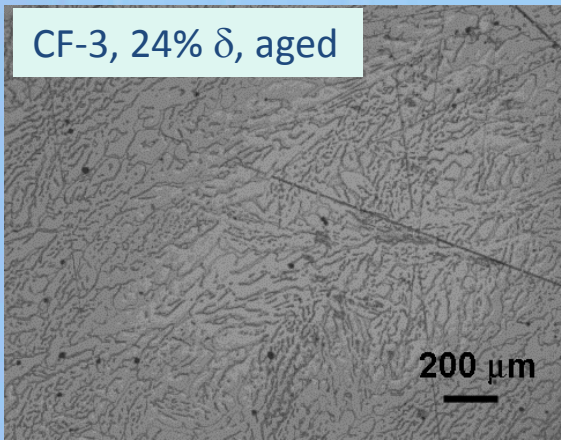
CF-3, 24% δ , un-aged



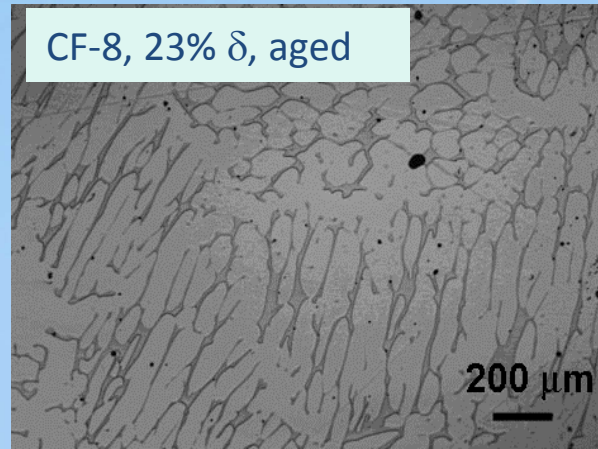
CF-8, 23% δ , un-aged



CF-3, 24% δ , aged

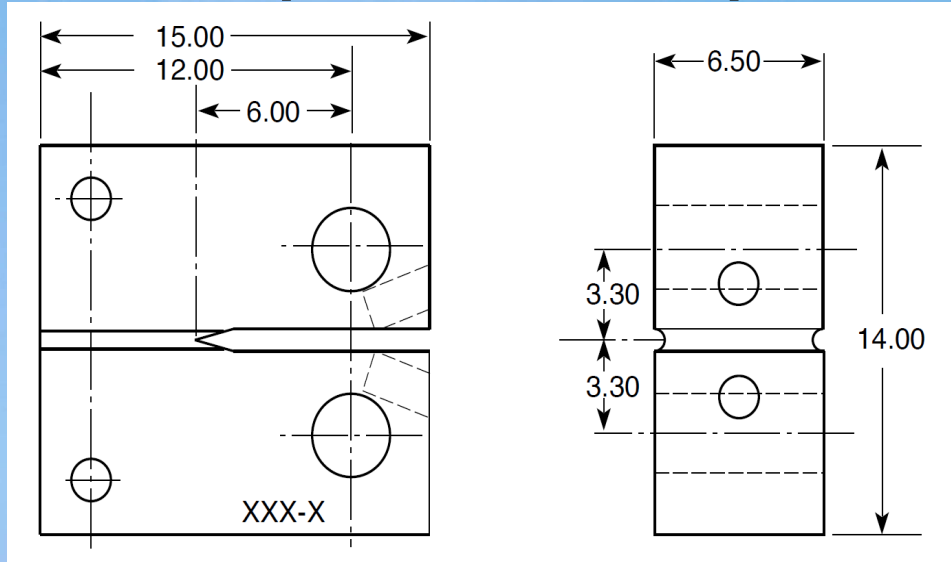


CF-8, 23% δ , aged

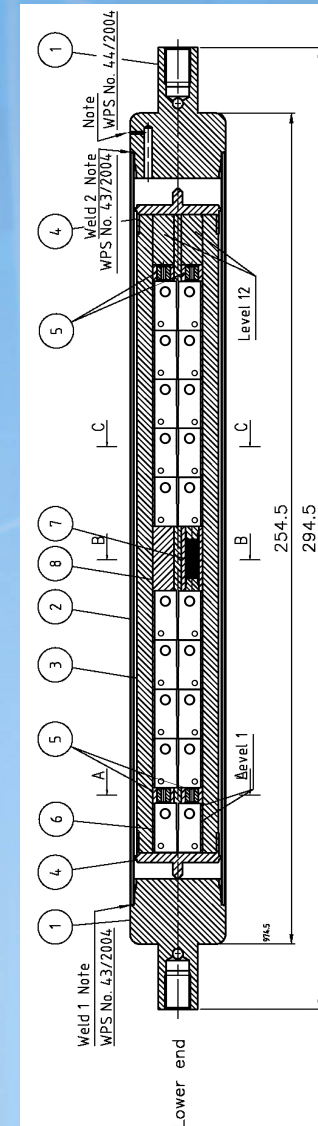


Specimens and Irradiation

- 1/4T-Compact Tension specimens**



- Irradiated in helium-filled capsule at Halden**
 - Irradiation temperature: ~315°C
 - Dose rate: 10^{-7} dpa/s,
 - Dose: $5.0E19$ n/cm² (E>1MeV), 0.08 dpa.

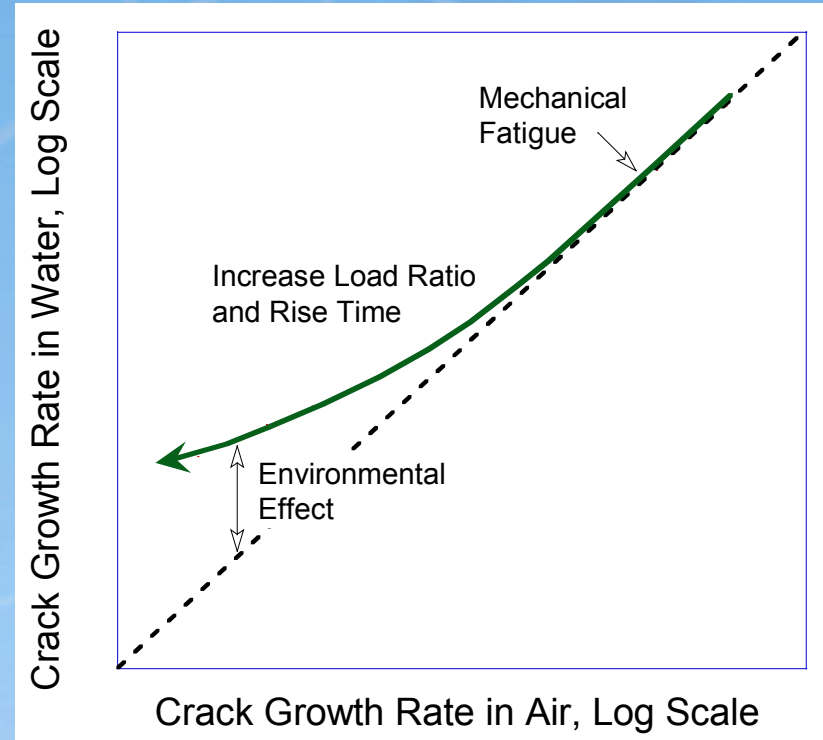


■ CGR tests:

- Precrack in test environments, and transition to SCC test.
- Constant-load CGR tests with and without periodic partial unloading (PPU).

■ JR curve tests:

- Use SCC starter cracks, and test in the environments at $\sim 320^{\circ}\text{C}$.
- A very slow deformation rate, $\sim 0.43 \mu\text{m/s}$, is used.
- A blunting line of $J/4\sigma_f$ is used in the analyses.
- Limited by sample size, the J values determined cannot be validated for J_{IC} .



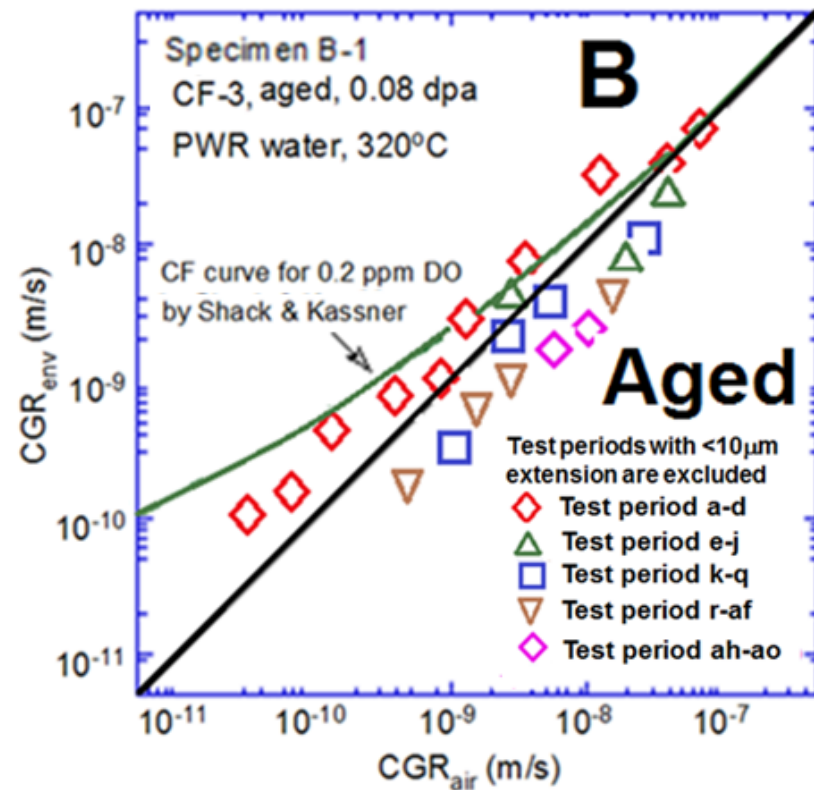
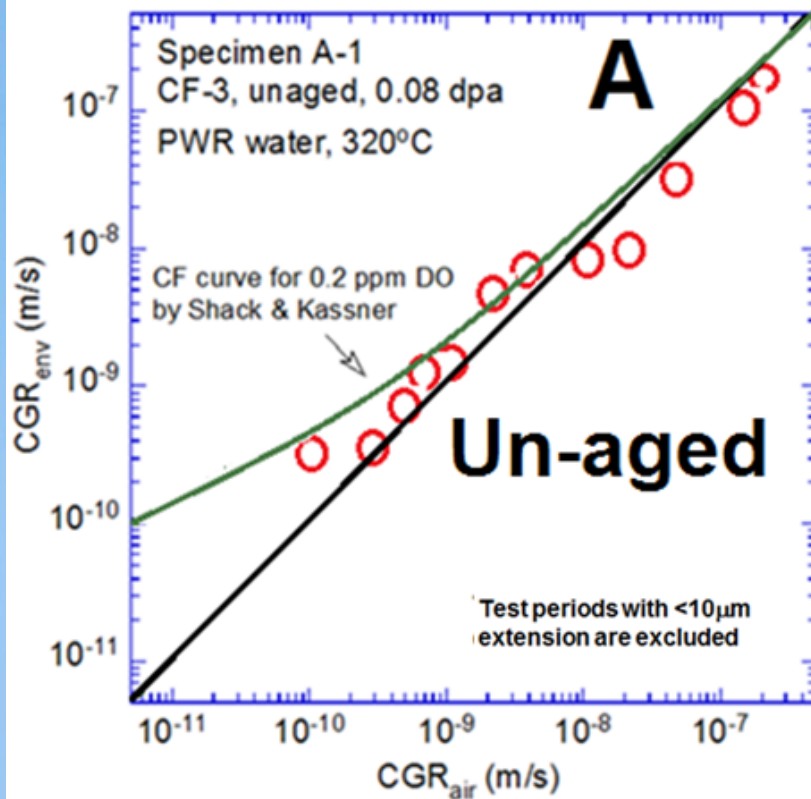


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CGR Test Results for un-aged and Aged CF-3 CASS Irradiated to 0.08 dpa



Crack growth rate test for CF-8 (A) un-aged and (B) aged CASS irradiated to dose of 0.08 dpa. Test environment and temperature PWR water, 320°C.

Note: Possible reasons for CGR data below the divide line

1. Test error
2. Crack may not be straight or crack surface has multiple contacts. Therefore DCPD did not respond correctly
3. Crack was arrested

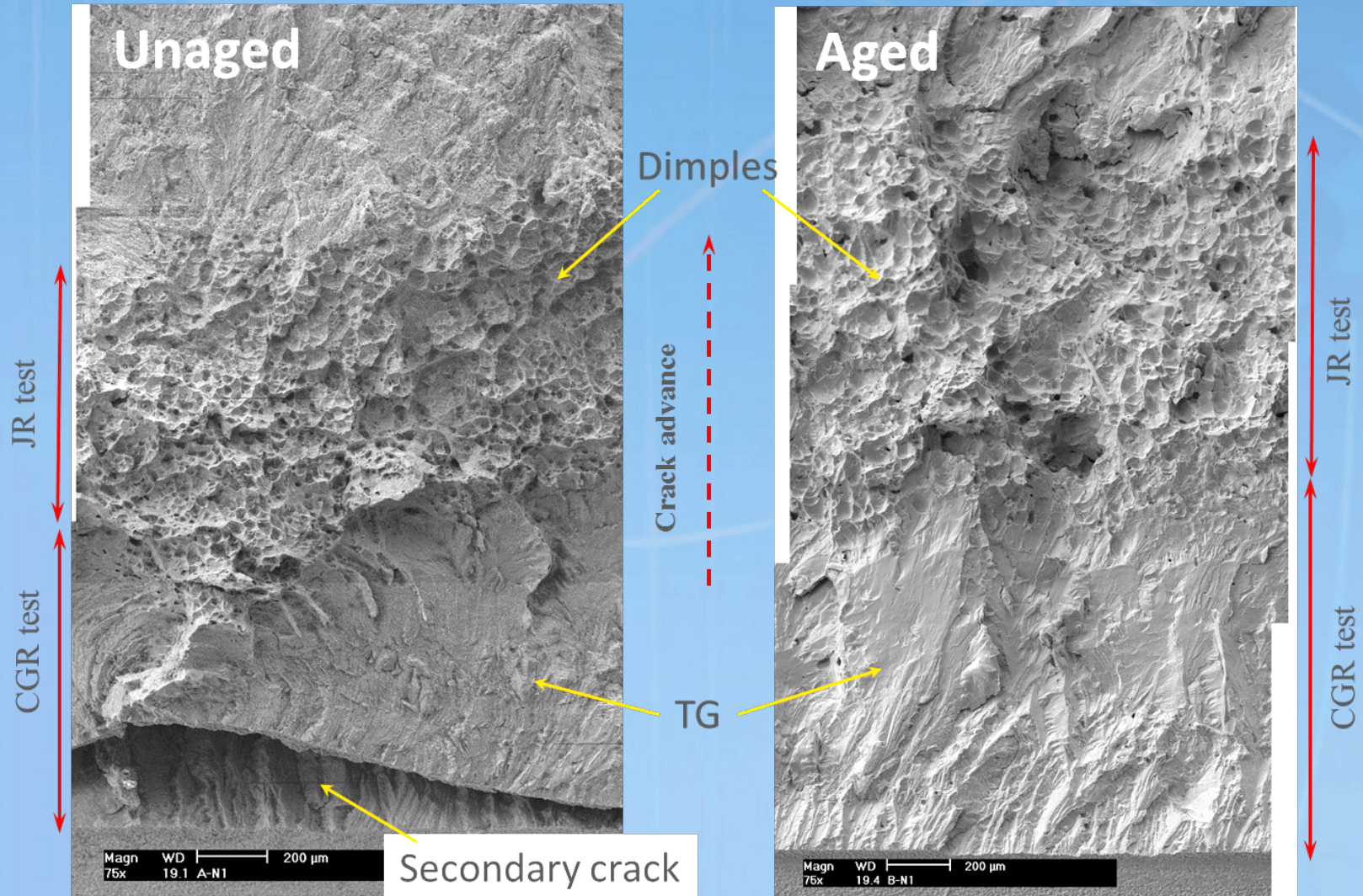


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Fracture Surface of Unirradiated - Unaged and Aged CF- 3 CASS Samples,



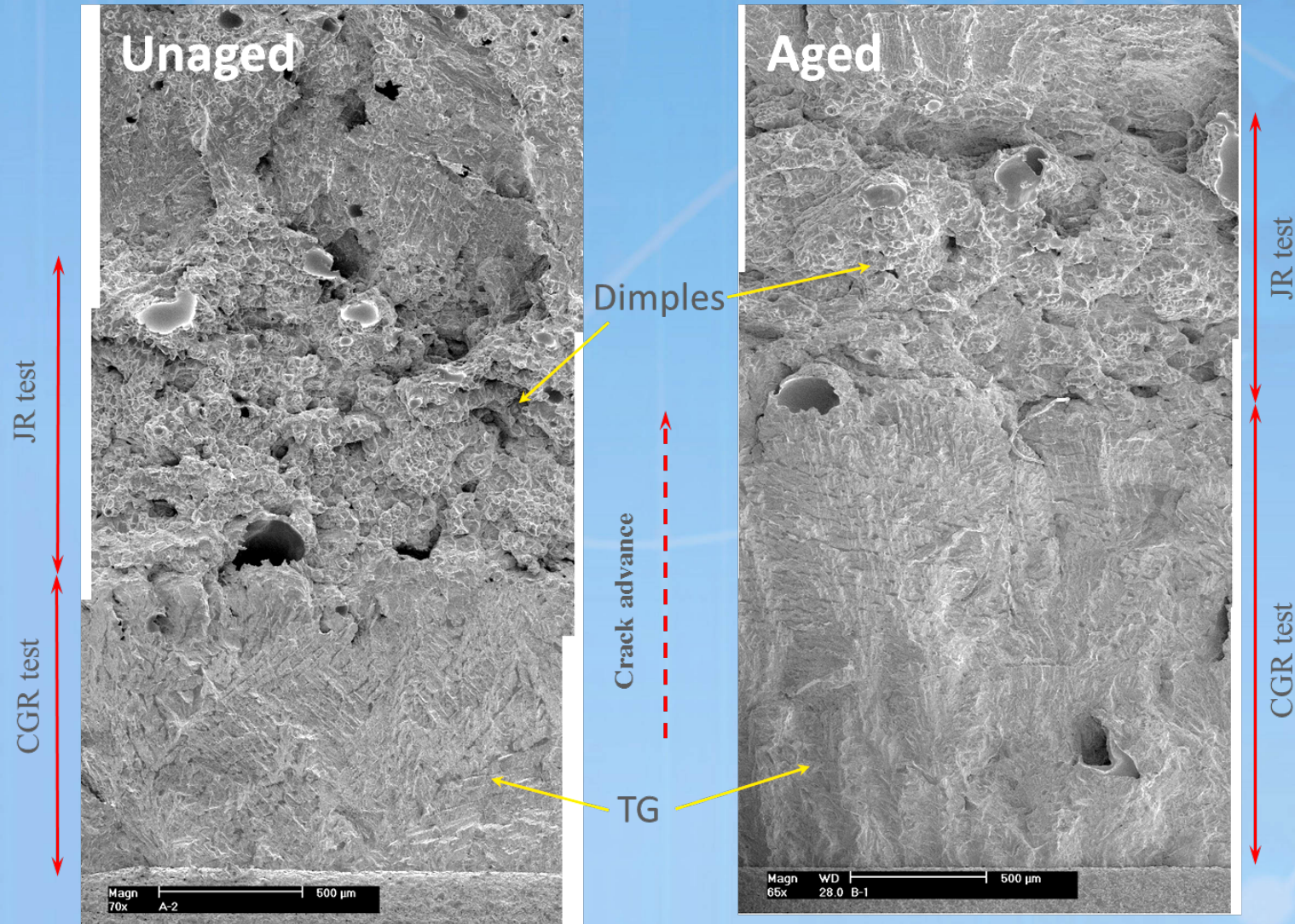


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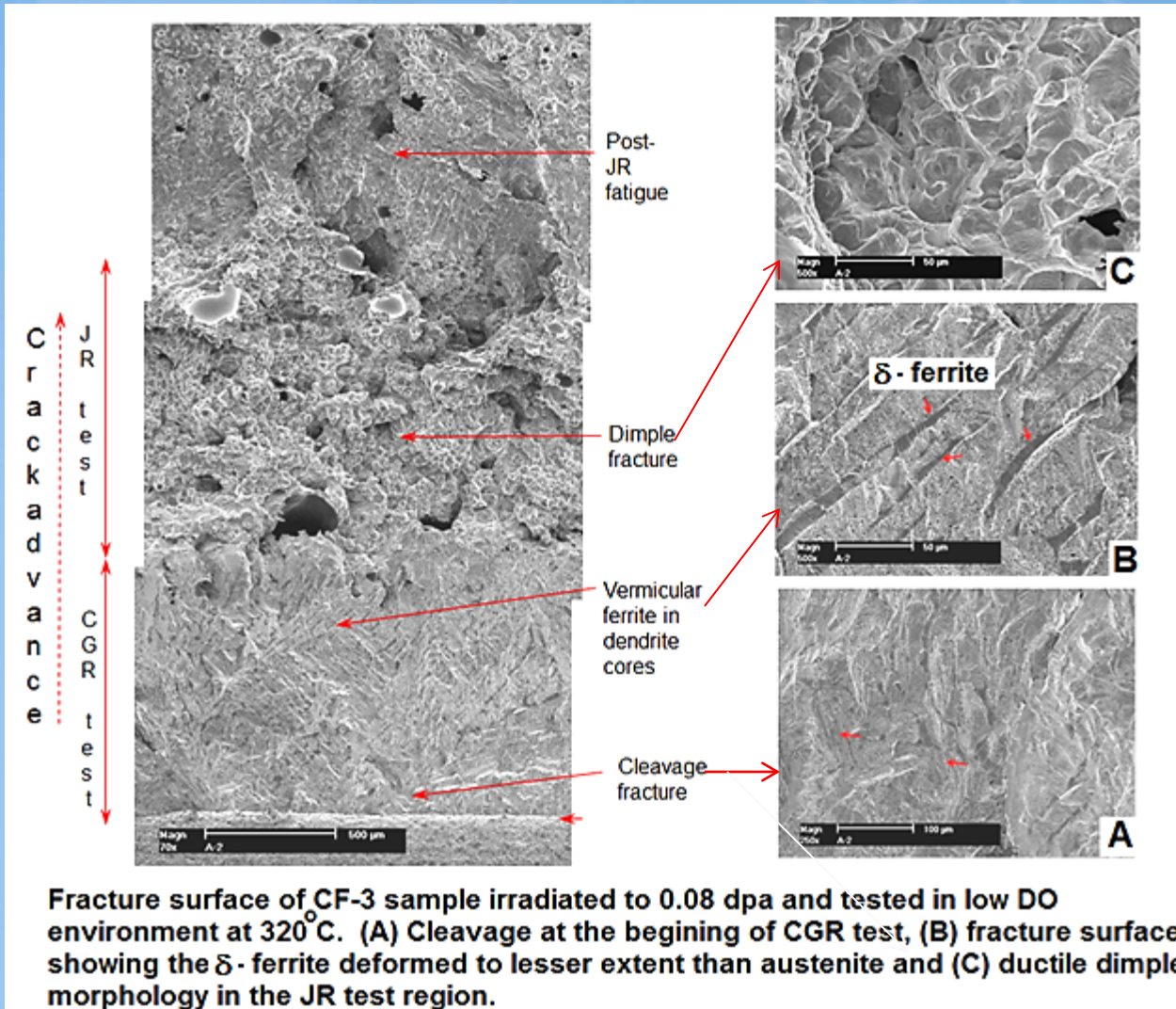
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Fracture Surface of irradiated (0.08 dpa) - Unaged and Aged CF- 3 CASS Samples,





Fracture surface of unaged CF- 3 CASS sample irradiated to 0.08 dpa



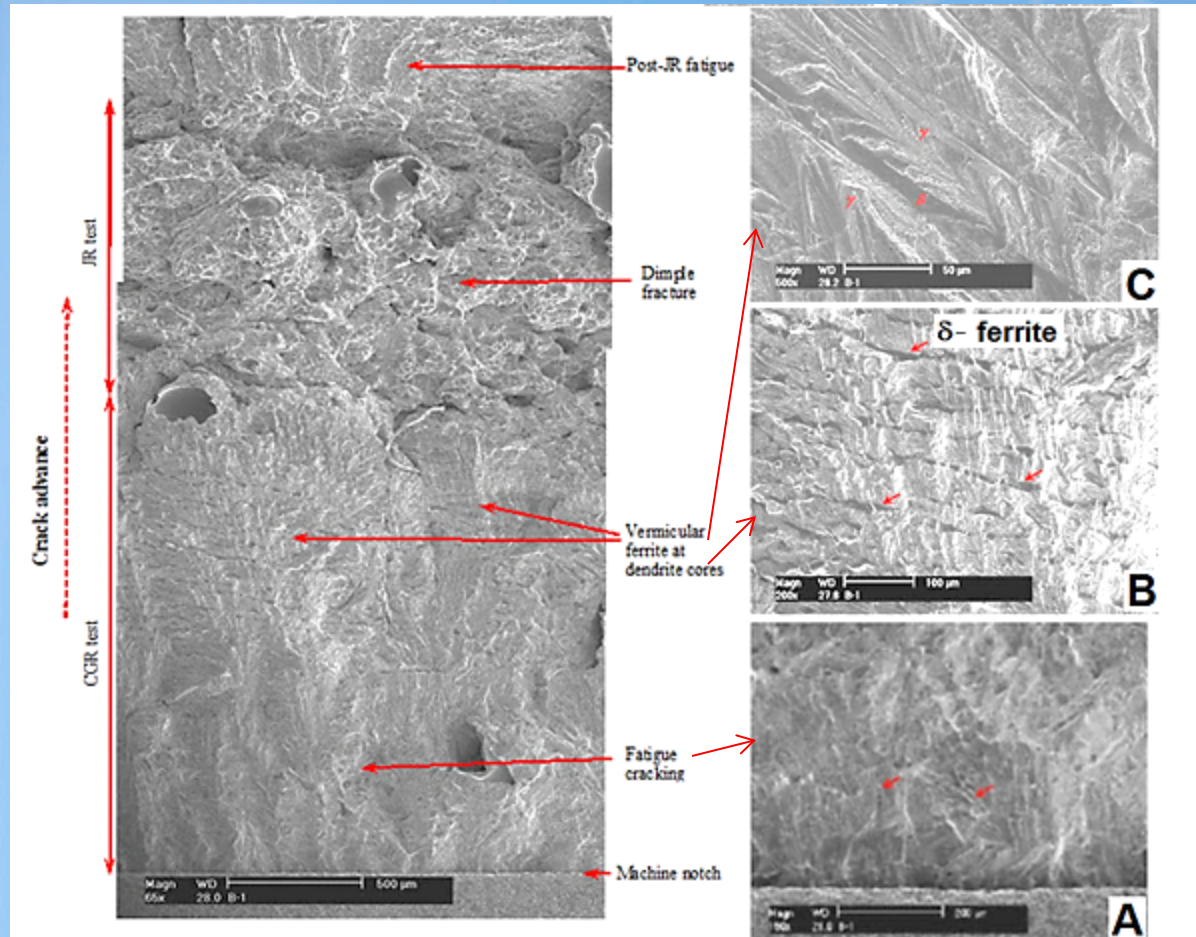


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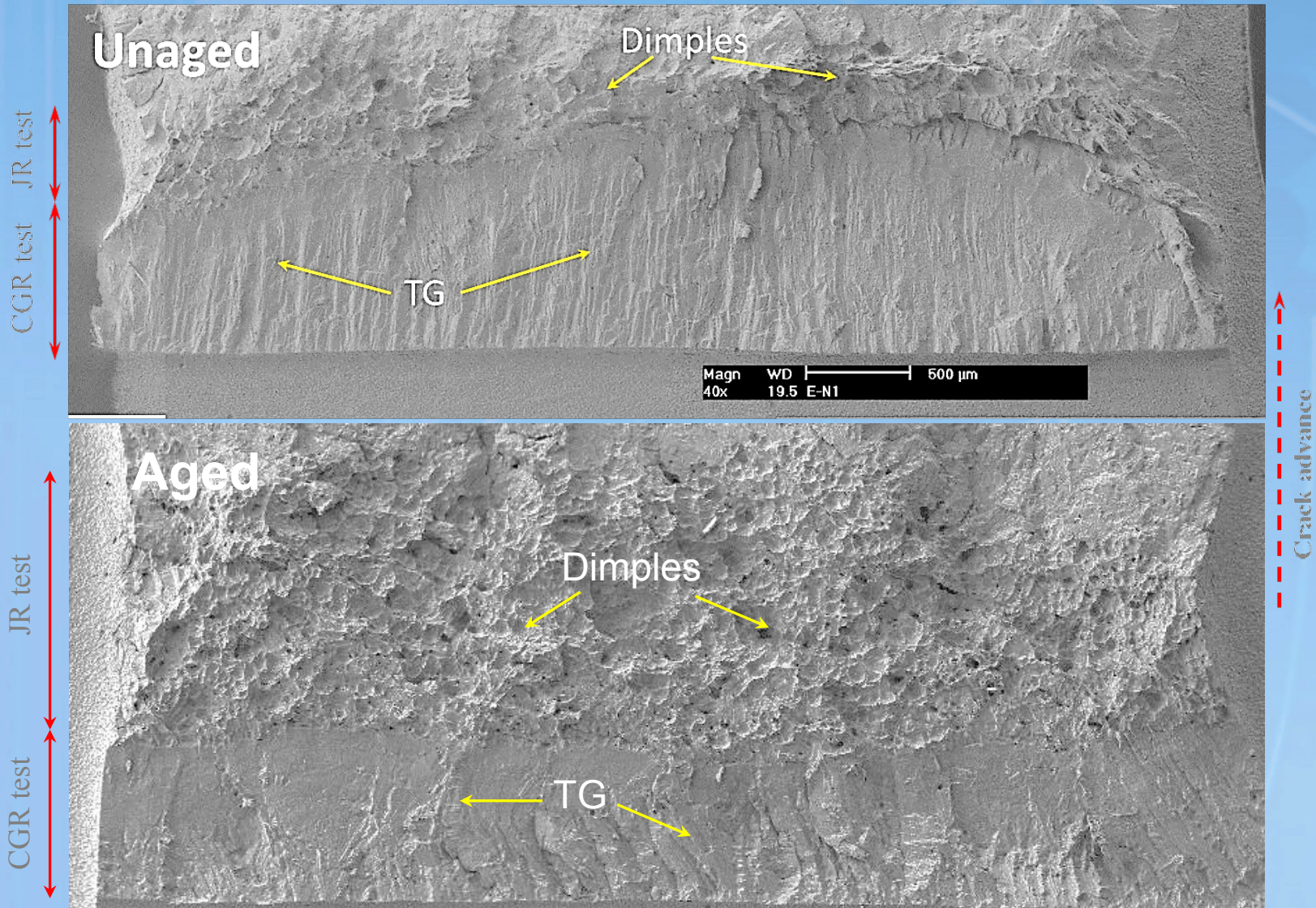
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Fracture surface of **aged** CF- 3 **CASS** sample irradiated to 0.08 dpa

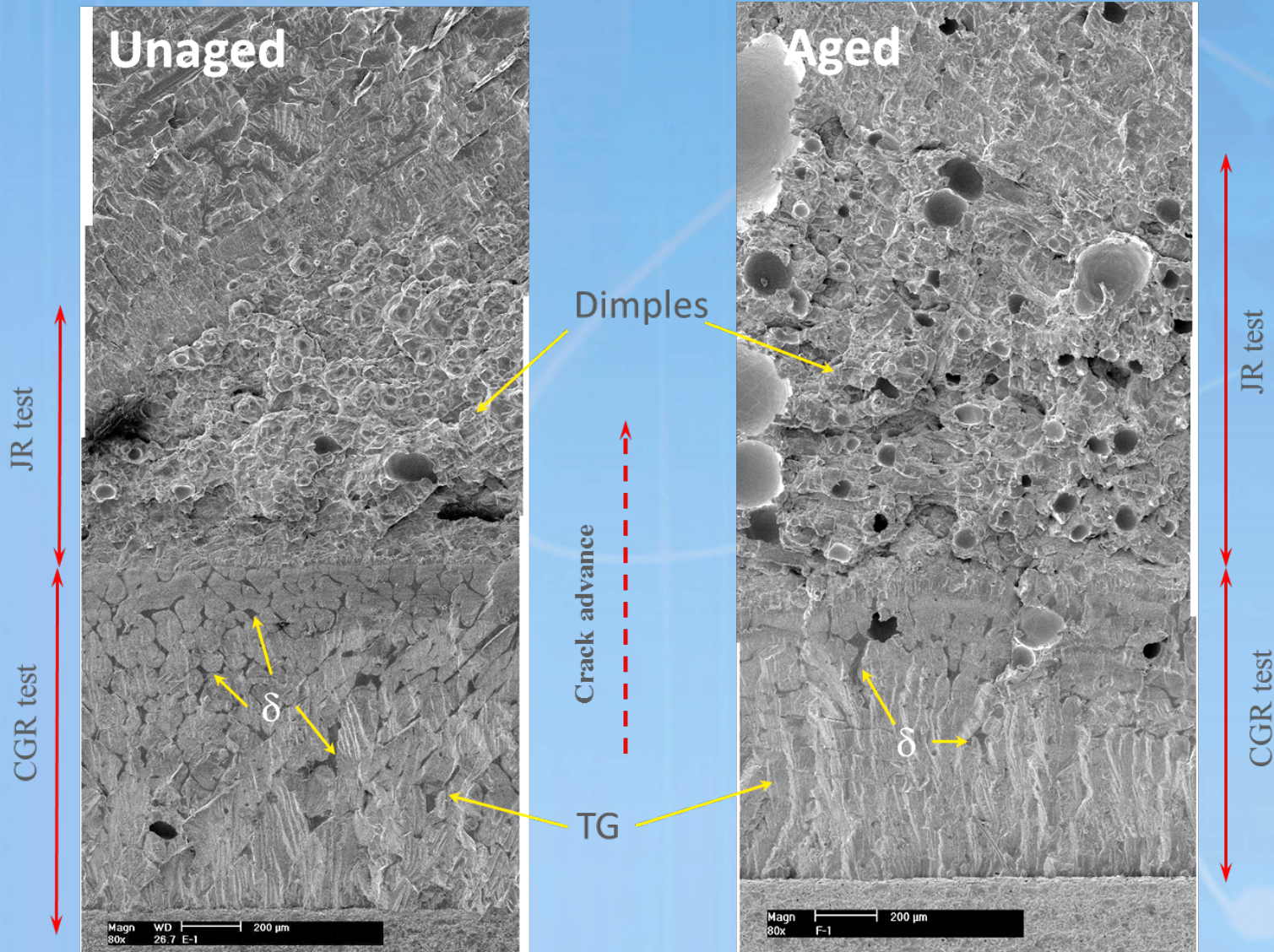


Fracture surface of aged CF-3 sample. Sample was aged at 400°C for 10,000 hours and was irradiated to 0.08 dpa. (A) cleavage steps in pre-cracking, (B) δ -ferrite dendrites core and (C) δ -ferrite is surrounded by heavily deformed austenite.

Fracture Surface of Unirradiated - Unaged and Aged CF- 8 CASS Samples,

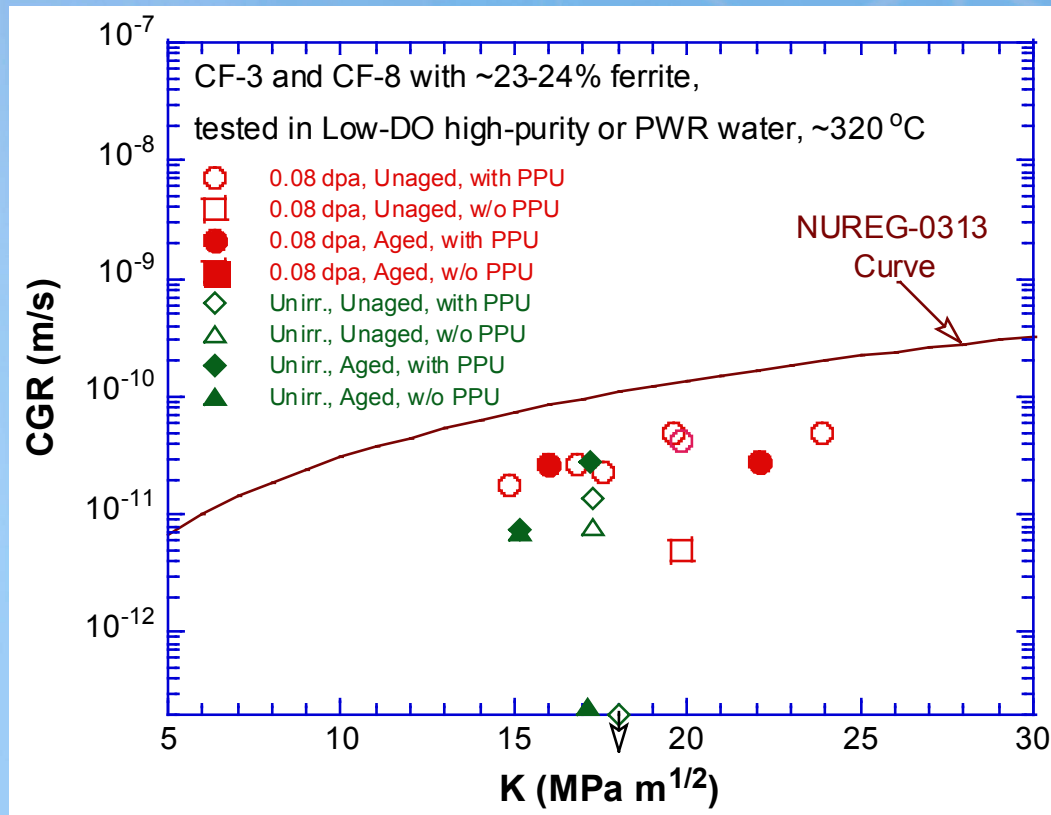


Fracture Surface of irradiated (0.08 dpa) - Unaged and Aged CF- 8 CASS Samples,





SCC CGR Results



- Neutron irradiation (~0.08 dpa) does not appear to elevate the cracking susceptibility significantly in low-corrosion-potential environments.
- Good SCC resistance for both aged and unaged CASS and are both below NUREG-0313 line. *Note: some data points have not been corrected.*

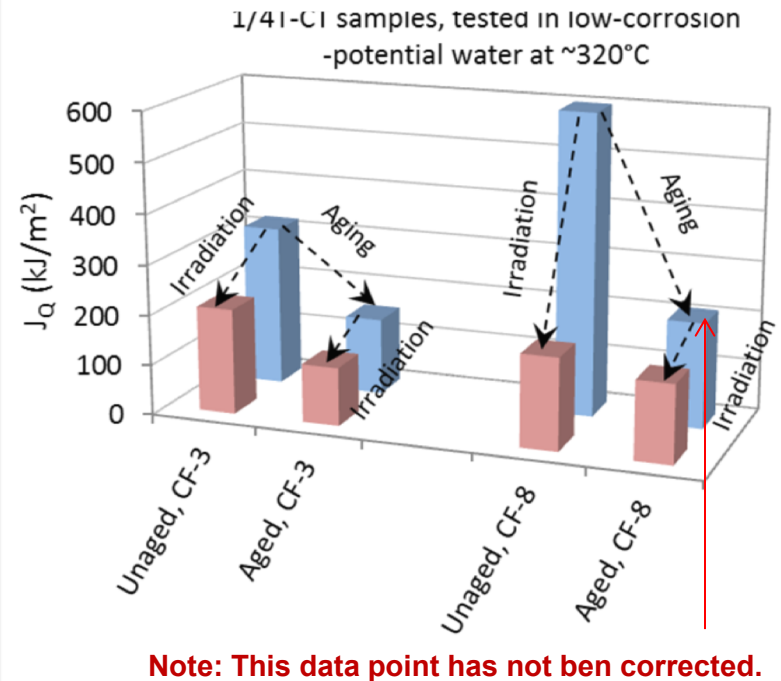
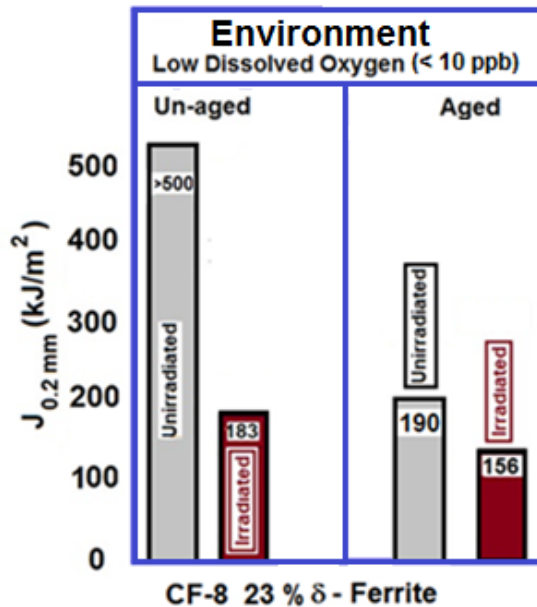
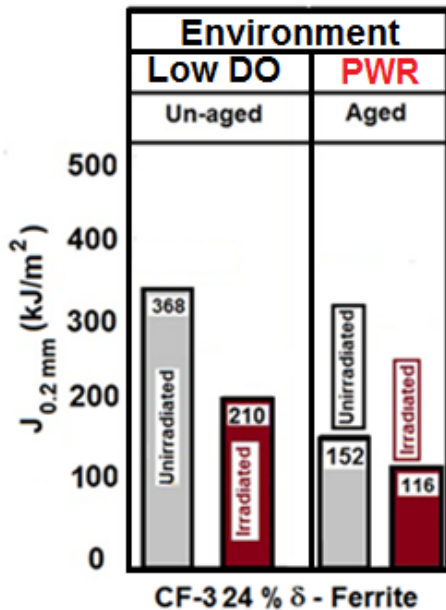


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Fracture Toughness



- Thermal aging effect on the fracture toughness more significant for CF-8 than CF-3 CASS.
- The irradiation embrittlement of both CF-3 and CF-8 CASS is greater for un-aged materials than for thermally aged materials.
- A combined effect of thermal aging and irradiation damage can reduce the fracture resistance of CASS further.

Summary

- **Cracking behaviors of thermally aged and unaged CASS are similar in both unirradiated and irradiated samples.**
- **All CASS specimens investigated here failed in a ductile (dimple) fracture mode during the fracture toughness /JR curve tests.**
- **Neutron irradiation, even at 0.08 dpa, had a significant impact on the fracture toughness under low-corrosion-potential environments of CASS.**
- **The fracture toughness of both CF-3 and CF- 8 CASS is higher in un-aged condition than in thermally aged condition.**
- **A combined effect of thermal aging and neutron irradiation is further decrease in the fracture resistance of both CF-3 and CF-8 CASS.**