



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 30, 2014

Vice President, Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 –STAFF SUMMARY OF
THE STEAM GENERATOR TUBE INSERVICE INSPECTIONS FOR THE
SPRING 2013 REFUELING OUTAGE (TAC NO. MF2614)

Dear Sir or Madam:

By letters dated August 15 and October 29, 2013 (Agencywide Documents Access and Management Systems Accession Number [ADAMS] ML13235A047 and ML13308B907, respectively), Entergy Nuclear Operations, Inc., the licensee, submitted information summarizing the results of the 2013 steam generator tube inspections performed at Indian Point Nuclear Generating Unit No. 3. Additional information concerning the 2013 steam generator tube inspections is included in the Nuclear Regulatory Commission letter dated April 16, 2013 (ADAMS ML13102A044).

The staff of the Steam Generator Tube Integrity and Chemical Engineering Branch of the Office of Nuclear Reactor Regulation has completed its review of these reports and concludes that the licensee provided the information required by the Indian Point Technical Specifications and that no additional follow-up is required. The Staff Summary is enclosed.

Sincerely,

A handwritten signature in black ink that reads "Douglas V. Pickett".

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure:
Staff Summary

cc w/encl: Distribution via Listserv



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INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
STAFF SUMMARY OF THE STEAM GENERATOR TUBE
INSERVICE INSPECTIONS FOR THE SPRING 2013 REFUELING OUTAGE
OFFICE OF NUCLEAR REACTOR REGULATION
DOCKET NO. 50-286

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The four steam generators at Indian Point 3 were replaced in 1989 with Westinghouse model 44F steam generators. Each steam generator contains 3214 thermally treated Alloy 690 tubes. Each tube has a nominal outside diameter of 0.875-inch and a nominal wall thickness of 0.050-inch. The tubes were hydraulically expanded at both ends for the full length of the tubesheet and are supported by a number of stainless steel tube support plates. The tubes installed in rows 1 through 8 were thermally stress-relieved after bending.

The licensee provided the scope, extent, methods, and results of their steam generator tube inspections in the documents referenced above. In addition, the licensee described corrective actions (e.g., tube plugging) taken in response to the inspection findings.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by the Technical Specifications. In addition, the staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

Principal Contributor: C. Hunt

Date: January 30, 2014

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/ra/

Douglas V. Pickett, Senior Project Manager
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Office of Nuclear Reactor Regulation

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ADAMS ACCESSION NO.: ML14015A417

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