

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-361 San Onofre Nuclear Station, Unit 2, Southern California 05000361
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SUBJECT: Suppls info in 820911 ltr requesting approval of Proposed
 Change NPF-10-42 to License NPF-10, changing implementation
 post-accident sampling sys to 830101, Interim post-accident
 sampling capability info encl.

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K. P. BASKIN
MANAGER OF NUCLEAR ENGINEERING,
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September 14, 1982

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Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
Licensing Branch No. 3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-361
San Onofre Nuclear Generating Station
Unit 2

By letter dated September 11, 1982 Southern California Edison Company (SCE) requested NRC approval of Proposed Change NPF-10-42 to Facility Operating License NPF-10 for San Onofre Nuclear Generating Station, Unit 2. Proposed Change NPF-10-42 changes the implementation date of the Post Accident Sampling System from "prior to exceeding five (5) percent power" to January 1, 1983. The purpose of this letter is to supplement the information provided by the September 11, 1982 letter.

The primary purposes of the PASS are to (1) assess core conditions and (2) assess containment hydrogen level following occurrence of a significant accident. The PASS functions which provide the bulk of the information needed to assess core conditions are the Reactor Coolant System (RCS) liquid indications. Although SCE will not declare the RCS-liquid functions of PASS operable until the remainder of the system has been modified, in the interim SCE will maintain the RCS functions available for service as much as is possible consistent with the need to take those functions out of service to work on the remainder of the system. One criterion of planning the subject modifications will be maximizing availability of the RCS liquid sample function. It is estimated that these functions will be available about seventy-five percent of the time. Indication of containment atmosphere hydrogen levels is provided by the hydrogen monitors which are fully operable.

The secondary functions of PASS (containment atmosphere, containment sump and RCS-gas) are addressed in Enclosure 1. In addition, as modifications of these PASS functions are completed, they will be maintained available as much as possible consistent with the completion of other PASS modifications.

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PDR

Mr. Frank Miraglia

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As stated in the September 11, 1982 letter, approval of Proposed Change NPF-10-42 is needed urgently. If you have any questions concerning the enclosed information, please contact me.

Very truly yours,

VP Baslam

cc: H. Rood, To be opened by addressee only

ENCLOSURE 1

INTERIM POST ACCIDENT SAMPLING CAPABILITY

PASS FUNCTION	STATUS	INTERIM CAPABILITY
I. Containment Atmosphere		
A. Hydrogen	Not Available	Hydrogen will be detected and measured by means of the Seismic I, Quality Class II, NUREG-0588 qualified IE powered containment hydrogen monitors. These instruments provide redundant, channelized continuous readout of hydrogen in containment in concentrations from 0 to 10%. One channel is recorded. Presence of hydrogen in the containment atmosphere will be indicative of core degradation, thus providing a measure of hydrogen production of a degraded core resulting from core voiding and metal-water reaction.
B. Radionuclide	Not Available	<p>The normal sampling system would be utilized during most of the FSAR Chapter 15 postulated spectrum of accidents because radiation levels would not preclude these samples being processed in the sample lab. In addition the dose rates at the high range in containment area radiation monitors have been correlated to reactor coolant activity levels as shown in FSAR Figures 432.42-1 and 432.42-2. Curves of dose rate versus time are provided for the following conditions:</p> <p>RCS Average - 100% of core average activity from FSAR Table 11.1-3 released to containment.</p> <p>RCS Maximum - 100% of core average activity from FSAR Table 11.1-2 released to containment.</p>

PASS FUNCTION

STATUS

INTERIM CAPABILITY

1% Failed Fuel - Assumes that 1% of total core activity is available for release and of that 1%, 100% of the noble gases, 50% of the halogens and 1% of the other isotopes are released.

10% Failed Fuel: 10 times the 1% case.

Gap Activity: Utilizes Reg. Guide 1.25 assumptions with 10% of the core Xenon and Krypton, 30% of the Krypton 85 and 10% of the Iodines released.

LOCA: Utilizes Reg. Guide 1.4 assumptions with 100% of the noble gases, 50% of the halogens and 1% of the other isotopes released.

Extrapolation between these curves will provide an indication of degraded core conditions.

C. Diluted samples

Not Available

Normal sampling is available as discussed above for Radionuclides in IB.

II. RCS Analysis

A. Gas

1. Hydrogen

Not Available

Use Post LOCA Hydrogen monitor. See discussion in I.A. above.

2. Oxygen

Not Available

Oxygen measurements are not a requirement of NUREG-0737 but are useful in analyzing the combined effects of chlorides on reactor coolant boundary materials.

3. Radionuclide

Not Available

Use the high range incontainment monitors. See the discussion of I.B.

PASS FUNCTION

STATUS

INTERIM CAPABILITY

4. Total Dissolved
Gas

Not Available

The Seismic I, Quality Class II, IE, NUREG-0588 qualified subcooled margin monitor and the interim core exit thermocouple system for detection of ICC will provide temperature and pressure parameters necessary to determine the amount of dissolved gas which reactor coolant can retain.

B. Liquid

1. Boron, pH, Radio- Available
nuclide

III. Containment Sump

Not Available

The analysis of containment sump liquids may be correlated with the RCS sample, corrected by the amount of spray and safety injection water. The amount of water injected and the RCS inventory are available from safety grade Refueling Water Storage Tank and Safety Injection Tank level indications. The activity of the RCS coolant is known from the PASS RCS liquid sample.