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 FACIL: 50-361 San Onofre Nuclear Station, Unit 2, Southern Californ 05000361
 50-362 San Onofre Nuclear Station, Unit 3, Southern Californ 05000362
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 RECIP. NAME RECIPIENT AFFILIATION
 MIRAGLIA, F. Licensing Branch 3

SUBJECT: Forwards "Pressurizer Safety Valve Operability & Safety
 Valve Discharge Piping Adequacy Rept," as final evaluation
 of EPRI/C-E program test results. Info satisfies Condition
 (19)j of License NPF-10.

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K. P. BASKIN
MANAGER OF NUCLEAR ENGINEERING,
SAFETY, AND LICENSING

June 29, 1982

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Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
Licensing Branch No. 3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Docket Nos. 50-361 and 50-362
San Onofre Nuclear Generating Station
Units 2 and 3

Item II.D.1, Relief and Safety Valve Testing, of NUREG-0737 as clarified by the NRC's letter of September 29, 1981 required Pressurized Water Reactor (PWR) Licensees to submit a preliminary evaluation of test results which demonstrate the capability of relief and safety valves to operate under expected operating and accident conditions by April 1, 1982 and to submit a final evaluation including analysis of discharge piping systems by July 1, 1982. This requirement has been identified as condition (19)j of Operating License No. NPF-10 for San Onofre Unit 2.

Consistent with this requirement, SCE's letter of April 1, 1982 provided a preliminary evaluation of the Electric Power Research Institute and Combustion Engineering (EPRI/CE) program test results demonstrating the capability of the Dresser 31709 NA Safety Valves used at San Onofre Unit 2 and 3 to operate under expected operating and accident conditions.

Enclosed please find seven (7) copies of the "San Onofre Nuclear Generating Station, Units 2 and 3 Pressurizer Safety Valve Operability and Safety Valve Discharge Piping Adequacy Report" which provides a final evaluation of the EPRI/CE program test results and demonstrates, (1) satisfactory operability of the San Onofre Units 2 and 3 safety valves, and (2) the adequacy of safety valve discharge piping system (NRC Mail Code B028).

It should be noted that during the dynamic load analysis of the safety valve discharge piping system, based on the EPRI/CE test data, one of the snubbers in the discharge piping system was found to have loads exceeding the manufacturer's load rating. The rating was based on the original design static load calculations. Further analysis demonstrated that this snubber was not necessary for the adequacy of the discharge piping system during safety

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Mr. Frank Miraglia

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valve operation. Part C of the enclosed report which discusses the adequacy of the safety valve discharge piping system contains a comparison of the piping system with and without the snubber and demonstrates the adequacy of the discharge piping system, including pipe supports, snubbers and their structures to satisfactorily facilitate safety valve operation without the snubber. Accordingly, the snubber has been deleted from the San Onofre Units 2 and 3 safety valve discharge piping configuration.

SCE considers that the enclosed information satisfies the requirements of condition (19)j of Operating License No. NPF-10 for San Onofre Unit 2.

If you have any questions or comments concerning the enclosed information, please contact me.

Very truly yours,

RW Kueger
for KP Baskin

Enclosures

SEE RPI

Southern California Edison Company

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April 30, 1982

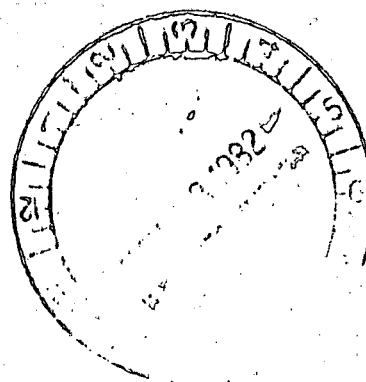
R P BASRIN

MANAGER OF NUCLEAR ENGINEERING
SAFETY AND ENVIRONMENT

Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
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Enclosed are sixty-three (63) copies of the response to the second part of the RC information request in the December 22, 1980 letter concerning compliance with NUREG-0612, Control of Heavy Loads (response to Sections 2.2, 2.3 and 2.4 of Enclosure 3 to the NRC December 22, 1980 letter). The response to the first part of the control of heavy loads information request for San Onofre Units 2 and 3 was submitted to the NRC by letter dated July 7, 1981. Additional information responding to Section 2.4 of Enclosure 3 of the December 22, 1980 letter will be provided by June 30, 1982 for several additional cranes which were either not in place or not expected to be permanent equipment when the July 7, 1981 report was submitted.

If you have any questions or comments concerning this information, please contact me.

Very truly yours,

R P Basrin

Enclosures

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