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 MIRAGLIA, F. Licensing Branch 3

SUBJECT: Forwards status &/or schedule for implementation of
 NUREG-0737, Items I.A.3.1, II.B.2, II.B.3, II.B.4, II.E.1.2,
 II.E.4.2 & II.F.1, in response to NRC 820317 ltr.

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April 15, 1982

Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
Licensing Branch No. 3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555



Gentlemen:

Subject: Docket No. 50-361
San Onofre Nuclear Generating Station
Unit 2

The NRC's letter of March 17, 1982 (Generic Letter No. 82-05) requested all licensees of operating power reactors to provide a status and/or schedule for selected NUREG-0737 items identified in Enclosure 1 of the NRC's letter pursuant to the requirements of 10 CFR 50.54(f).

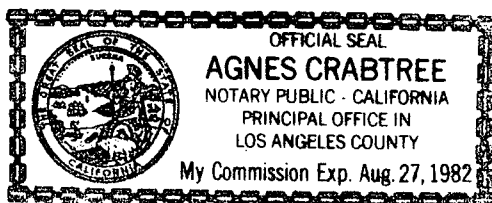
Consistent with the NRC's request, enclosed please find the current status and/or schedule of the NUREG-0737 items identified in Enclosure 1 of Generic Letter No. 82-05 which are applicable to San Onofre Unit 2.

If you have any questions or comments, please let me know.

Subscribed on this 15th day of April, 1982.

Very truly yours,

By M. O. Medford
M. O. Medford
Supervising Engineer
San Onofre Units 2 and 3 Licensing



Subscribed and sworn to before me
this 15th day of April, 1982

Agnes Crabtree
Notary Public in and for the County
of Los Angeles, State of California

Enclosures

8204160331 820415
PDR ADOCK 05000361
PDR

Boo
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STATUS AND/OR SCHEDULE OF SELECTED NUREG-0737 ITEMS
IDENTIFIED IN ENCLOSURE 1 OF NRC GENERIC LETTER NO. 82-05
WHICH ARE APPLICABLE TO SAN ONOFRE UNIT 2

Item	Title	Status/Schedule	Discussion/Justification
I.A.3.1	Simulator Exams	N/A	Simulator exams were not a requirement for License Examinations which were administered to the initial San Onofre Unit 2 Cold License Candidates on January 22, 1982.
II.B.2	Plant Shielding	Complete	In Supplement No. 1 to the San Onofre Units 2 and 3 Safety Evaluation Report (SER) dated February, 1981, the NRC Staff indicated that the San Onofre Units 2 and 3 design met the requirements of NUREG-0737, Item II.B.2.
II.B.3	Post-Accident Sampling	Prior to operation above 5% power	SCE's letter of December 22, 1981 requested schedular relief prior to operation above 5% power with the justification that sufficient fission product inventory to warrant the use of this system in the event of an accident will not exist prior to operation above 5% power. Condition (19)i of Operating License No. NPF-10 for San Onofre Unit 2 accurately reflects the "prior to exceeding 5% power" schedule.
II.B.4	Training for Mitigating Core Damage	Complete	This training was provided to operators as part of their normal training program which was completed prior to administration of License Examinations in January, 1982.
II.E.1.2	Aux. Feedwater Initiation and Flow Indicator	Complete	In Supplement No. 1 to the San Onofre Units 2 and 3 SER, the NRC Staff indicated that the San Onofre Units 2 and 3 design met the requirements of NUREG-0737, Item II.E.1.2.
II.E.4.2	Containment Isolation Dependability		
	Position 5 - Lower containment pressure setpoint to level compatible with normal operation.	Complete	In Supplement No. 1 to the San Onofre Units 2 and 3 SER, the NRC Staff indicated that the San Onofre Units 2 and 3 design met all the requirements of NUREG-0737, Item II.E.4.2, including Positions 5 and 7.
	Position 7 - Isolate purge and vent valves on radiation signal	Complete	

Item	Title	Status/Schedule	Discussion/Justification
II.F.1	Accident Monitoring		
	Attachment 1 - Install noble gas effluent monitors	Prior to operation above 5% power	SCE's letter of December 22, 1981 requested schedular relief from the requirement of Attachments 1 and 2 to prior to operation above 5% power with the justification that sufficient fission product inventory to warrant the use of these systems in the event of an accident will not exist prior to operation above 5% power. Condition (19)n of Operating License No. NPF-10 for San Onofre Unit 2 accurately reflects the prior to exceeding 5% power schedule.
	Attachment 2 - Provide capability for effluent monitoring of iodine	Prior to operation above 5% power	
	Attachment 3 - Install in-containment radiation-level monitor	Complete	The requirements of Attachment 3 of item II.F.1 were verified to be complete in NRC Inspection Report 50-361/81-35 dated February 19, 1982.
	Attachment 4 - Provide continuous indication of containment pressure	Complete	The requirements of Attachments 4 and 5 of Item II.F.1 were verified to be complete in NRC Inspection Report 50-361/82-04 dated January 18, 1982.
	Attachment 5 - Provide continuous indication of containment water level	Complete	
	Attachment 6 - Provide continuous indication of hydrogen concentration in containment	Complete	The requirements of Attachment 6 of Item II.F.1 were verified to be complete in NRC Inspection Report 50-361/82-10 dated February 12, 1982.