

REGULARY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8206100237 DOC.DATE: 82/05/20 NOTARIZED: NO DOCKET #
 FACIL:50-361 San Onofre Nuclear Station, Unit 2, Southern Californ 05000361
 AUTH.NAME AUTHOR AFFILIATION
 BASKIN,K.P. Southern California Edison Co.
 RECIP.NAME RECIPIENT AFFILIATION
 MIRAGLIA,F. Licensing Branch 3

SUBJECT: Forwards requests for NRC approval to revise proposed changes to Tech Spec 3.3.2, Table 3.3-5 re response time testing for auxiliary feedwater train. Formal amend application will be submitted by 820531.

DISTRIBUTION CODE: 8021S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 244
 TITLE: Correspondence for Operating & Non-Operating Facility (e.g. North Ann

NOTES: J Hanchett 1cy PDR Documents. ELD Chandler 1cy. 05000361
 NRR Scaletti 1cy.

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
LIC BR #3 BC 05	1 1	LIC BR #3 LA	1 0
ROOD,H. 01	1 1		
INTERNAL: ELD/HDS2	1 0	IE/DEP EPDS	1 1
IE/DEP/EPLB	3 3	MPA	1 0
NRR/DE/CEB 09	1 1	NRR/DE/eqb 14	1 1
NRR/DE/GB 15	4 4	NRR/DE/HGEB 17	1 1
NRR/DE/MEB 20	1 1	NRR/DE/MTEB 19	1 1
NRR/DE/QAB 23	1 1	NRR/DE/SAB 26	1 1
NRR/DE/SEB 27	1 1	NRR/DHFS DEPY12	1 1
NRR/DHFS/HFEB16	1 1	NRR/DHFS/LQB	1 1
NRR/DHFS/OLB	1 1	NRR/DHFS/PTRB22	1 1
NRR/DL DIR	1 0	NRR/DL/ORAB	1 0
NRR/DSI/ADRS	1 0	NRR/DSI/AEB 28	1 1
NRR/DSI/ASB 08	1 1	NRR/DSI/CPB 11	1 1
NRR/DSI/CSB 10	1 1	NRR/DSI/ETSB 13	1 1
NRR/DSI/ICSB 18	1 1	NRR/DSI/PSB 21	1 1
NRR/DSI/RAB 24	1 1	NRR/DSI/RSB 25	1 1
NRR/DST/LGB	1 1	REG FILE 04	1 1
RGNS	1 1		
EXTERNAL: ACRS 29	10 10	FEMA-REP DIV	1 1
LPDR 03	1 1	NRC PDR 02	1 1
NSIC 06	1 1	NTIS	1 1
NOTES:	3 3		

TOTAL NUMBER OF COPIES REQUIRED: LTTR 59 ENCL 53

Southern California Edison Company



P. O. BOX 800
2244 WALNUT GROVE AVENUE
ROSEMEAD, CALIFORNIA 91770

K. P. BASKIN
MANAGER OF NUCLEAR ENGINEERING,
SAFETY, AND LICENSING

May 20, 1982

TELEPHONE
(213) 572-1401

Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
Licensing Branch No. 3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-361
San Onofre Nuclear Generating Station
Unit 2

Enclosed for your review and approval is a copy of a Proposed Change NPF-10-19 to Technical Specification 3.3.2, Table 3.3-5, Item 8.a(2) and Item 9.a(2) for San Onofre Nuclear Generating Station, Unit 2, incorporated in Facility Operating License NPF-10 as Appendix A. The proposed change is a request to clarify that response time testing for Auxiliary Feedwater Train identified in Table 3.3-5 is not required for entry into Mode 3.

NRC approval of the proposed change is requested to facilitate the entry of Unit 2 into Operational Mode 3. A formal request for an amendment to Facility Operating License No. NPF-10 detailing this proposed change will be transmitted to the NRC during the week of May 31, 1982. The formal request will include a check in the amount of \$1,200.00 for this change which has been determined to be a Class II change in accordance with 10 CFR 170.22.

If you have any questions concerning the enclosed information, please call me.

Very truly yours,

M. D. Medford for KPB

Enclosure

3021

8206100237 820520
PDR ADCK 05000361
PDR

DESCRIPTION OF PROPOSED CHANGE NPF-10-19 AND SAFETY ANALYSIS

This is a request to revise Appendix "A" Technical Specification 3.3.2, Table 3.3-5.

ENGINEERED SAFETY FEATURES RESPONSE TIMES

EXISTING SPECIFICATION:

See Attachment "A"

PROPOSED SPECIFICATION:

Page Add to the end of Surveillance Requirement 4.3.2.3: "The provisions
3/4 3-13 of specification 4.0.4 are not applicable for entry into Mode 3 for
 items 8.a(2) and 9.a(2) of Table 3.3-5."

Page Add "(Note 6)" after items 8.a(2) and 9.a(2) of Table 3.3-5.
3/4 3-29

Page Add Note 6 as follows:
3/4 3-30

"6. The provisions of specification 4.0.4 are not applicable for entry into Mode 3."

SAFETY ANALYSIS:

The turbine driven auxiliary feedwater pump is not tested for operability, as specified in Specification 4.7.1.2.1.a.1, until after entry into Mode 3, when adequate steam pressure exists in steam generators to permit the pump to function as designed. This change is consistent with such requirements since proper determination of ESF response times for the turbine driven auxiliary feedwater pump is dependent upon having adequate steam pressure available.

Accordingly, it is concluded that: (1) Proposed Change NPF-10-19 does not involve an unreviewed safety question as defined in 10 CFR 50.59, nor does it present significant hazard considerations not described or implicit in the Final Safety Analysis; (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.