

NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-361
 UNIT NAME: SONGS - 2
 DATE: 4-15-91
 COMPLETED BY: M. M. Farr
 TELEPHONE: (714) 368-9787

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 2
2. Reporting Period: March, 1991
3. Licensed Thermal Power (MWt): 3390
4. Nameplate Rating (Gross MWe): 1127
5. Design Electrical Rating (Net MWe): 1070
6. Maximum Dependable Capacity (Gross MWe): 1127
7. Maximum Dependable Capacity (Net MWe): 1070
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7)
 Since Last Report, Give Reasons: NA
9. Power Level To Which Restricted, If Any (Net MWe): NA
10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.00</u>	<u>2,160.00</u>	<u>66,793.00</u>
12. Number Of Hours Reactor Was Critical	<u>677.43</u>	<u>2,093.43</u>	<u>48,852.99</u>
13. Reactor Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
14. Hours Generator On-Line	<u>677.43</u>	<u>2,093.43</u>	<u>47,895.85</u>
15. Unit Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,798,190.81</u>	<u>6,551,598.37</u>	<u>156,035,583.09</u>
17. Gross Electrical Energy Generated (MWH)	<u>583,941.50</u>	<u>2,213,128.00</u>	<u>52,920,622.00</u>
18. Net Electrical Energy Generated (MWH)	<u>548,679.00</u>	<u>2,104,353.00</u>	<u>50,156,875.24</u>
19. Unit Service Factor	<u>91.05%</u>	<u>96.92%</u>	<u>71.71%</u>
20. Unit Availability Factor	<u>91.05%</u>	<u>96.92%</u>	<u>71.71%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>68.92%</u>	<u>91.05%</u>	<u>70.18%</u>
22. Unit Capacity Factor (Using DER Net)	<u>68.92%</u>	<u>91.05%</u>	<u>70.18%</u>
23. Unit Forced Outage Rate	<u>8.95%</u>	<u>3.08%</u>	<u>6.05%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Cycle 6 Refueling outage scheduled to commence August 3, 1991, for a duration of 90 days.			
25. If Shutdown At End Of Report Period, Estimated Date of Startup:	<u>NA</u>		
26. Units In Test Status (Prior To Commercial Operation):	<u>Forecast Achieved</u>		

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

<u>NA</u>	<u>NA</u>
<u>NA</u>	<u>NA</u>
<u>NA</u>	<u>NA</u>

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-361
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MONTH: March 1991

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1	<u>1103.08</u>
2	<u>1005.17</u>
3	<u>1099.67</u>
4	<u>1074.58</u>
5	<u>1008.96</u>
6	<u>1104.42</u>
7	<u>1106.25</u>
8	<u>1101.67</u>
9	<u>805.08</u>
10	<u>362.79</u>
11	<u>0.00</u>
12	<u>0.00</u>
13	<u>276.29</u>
14	<u>962.88</u>
15	<u>1085.17</u>
16	<u>1078.75</u>

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17	<u>1082.13</u>
18	<u>1081.00</u>
19	<u>568.08</u>
20	<u>541.04</u>
21	<u>584.75</u>
22	<u>584.29</u>
23	<u>587.50</u>
24	<u>592.54</u>
25	<u>590.54</u>
26	<u>595.75</u>
27	<u>596.42</u>
28	<u>594.42</u>
29	<u>593.46</u>
30	<u>589.13</u>
31	<u>597.79</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-361

UNIT NAME: SONGS - 2

DATE: 4-15-91

COMPLETED BY: M. M. Farr

TELEPHONE: (714) 368-9787

REPORT MONTH: March 1991

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	LER No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
63	910309	S	0.00	B	5	NA	KE	COND	Reduced reactor power to 75% to support condenser water box cleaning.
64	910310	F	66.57	A	3	2-91-003	AA	CNTR	See note below.
65	910319	F	0.00	A	1	NA	NN	MO	Reduced reactor power to 60% due to overcurrent/ground on two Circulating Water Pumps. Currently awaiting repair of Circulating Water Pump Motors.

Note: The reactor automatically tripped from 77% power due to opening of the output contactor for Control Element Drive Mechanism Control System (CEDMCS) Motor Generator (MG) set while the other MG set was out of service for maintenance. This resulted in de-energization of the CEDMCS bus and an automatic reactor trip. The operational CEDMCS MG set ammeter (AMR) relay underexcitation contact was found to have a high resistance. This condition is concluded to have resulted in the opening of the output contactor. Upon subsequent testing of the relay, the contact failed open. The AMR relay was sent to an offsite laboratory for failure analysis. The failed ARM relay was replaced with an inkind part. The MG set was tested satisfactorily, and returned to service prior to restart of the plant. Appropriate corrective actions will be determined and implemented based on the results of the failure analysis of the AMR relay.

¹F-Forced
S-Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from
Previous Month
5-Reduction of 20%
or greater in the
past 24 hours
6-Other (Explain)

⁴IEEE Std 805-1984

⁵IEEE Std 803A-1983

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO: 50-361
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<u>Date</u>	<u>Time</u>	<u>Event</u>
March 1	0001	Unit is in Mode 1 at 100% reactor power. Turbine load at 1153 MWe gross.
March 2	0410	Commenced reactor power decrease to 80% for circulating water system heat treatment.
	0622	Reactor at 80% power.
	1300	Commenced reactor power increase to 100% following completion of heat treating operations.
	1634	Reactor at 100% power.
March 4	2000	Commenced reactor power decrease to 75% to support condenser water box cleaning.
	2330	Reactor at 75% power.
March 5	0510	Commenced reactor power increase to 85%.
	0625	Commenced reactor power increase to 100% following completion of condenser water box cleaning.
	0850	Reactor at 100% power.
March 8	2145	Commenced reactor power decrease to 75% to support condenser water box cleaning.
March 9	0405	Reactor at 75% power.
March 10	1206	Reactor tripped from 75% power due to loss of CEDMCS MG sets. Entered Mode 3.
March 12	2012	Entered Mode 2 following completion of repairs to CEDMCS MG sets.
	2057	Reactor is made critical.

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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<u>Date</u>	<u>Time</u>	<u>Event</u>
March 13	0235	Entered Mode 1.
	0640	Unit synchronized to the grid. Continued reactor power increase to 100% power.
March 15	0001	Reactor at 100% power.
March 19	0222	Circulating Water Pump P-117 tripped on overcurrent/ground. Commence rapid reactor power decrease to 75%.
	0233	Circulating Water Pump P-116 tripped on overcurrent/ground. Continue rapid reactor power decrease to 60%.
March 20	2400	Reactor at 60% power.
March 31	2400	Unit is in Mode 1 at 60% reactor power. Turbine load at 642 MWe gross. Awaiting repair of Circulating Water Pump Motors.

REFUELING INFORMATION

DOCKET NO:	<u>50-361</u>
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DATE:	<u>4-15-91</u>
COMPLETED BY:	<u>M. M. Farr</u>
TELEPHONE:	<u>(714) 368-9787</u>

MONTH: March 1991

1. Scheduled date for next refueling shutdown.

Cycle 6 refueling outage is forecast for August 1991.

2. Scheduled date for restart following refueling.

Restart from Cycle 6 refueling outage is forecast for November 1991.

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes.

What will these be?

There are several License Amendments required to support work being performed during the Unit 2 Cycle 6 refueling outage. A summary of each of these items follows:

- A. In accordance with SCE's letter dated October 31, 1990, SCE committed to remove the autoclosure interlock on the shutdown cooling valves. This commitment was a result of SCE's evaluation in accordance with Generic Letter 88-17. Specifically, Surveillance Requirement 4.5.2.d.1 will be revised to reflect removal of the automatic isolation of the shutdown cooling system from the reactor coolant system (RCS) when RCS pressure is greater than or equal to 715 psia. NRC approval will be required prior to returning the shutdown cooling system to Operable status during the outage. The work will be done during the core offload from day 20 to day 36 of the outage. Based on an August 3 start date, NRC approval will be required by about September 8, 1991.
- B. A change to the Technical Specifications and an exemption from 10CFR50, Appendix J is required to decouple the 10 year Inservice Testing from the ten year Integrated Leak Rate Test (ILRT). Specifically, Surveillance Requirement 4.6.1.2.a will be revised to remove the requirement that the third test of each set of ILRTs be conducted during the shutdown for the 10-year plant inservice inspection. NRC approval will be required prior to entry into Mode 4 at the end of the outage, anticipated about October 19.

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MONTH: March 1991

- C. NRC approval of changes to the UFSAR will be requested to permit use of the shutdown cooling system as the primary means of spent fuel pool cooling. Use of the shutdown cooling system will be required when it is necessary to perform maintenance on certain components in the component cooling water system and the spent fuel pool cooling system. Currently, a test of leakage of the cross train isolation valves in the component cooling water system will be performed at the beginning of the outage. If the leakage is excessive, NRC approval of these UFSAR changes will be required about 17 days into the outage, anticipated about August 20.
 - D. A change to the Technical Specifications will be requested to revise the snubber visual inspection frequency and criteria in accordance with Generic Letter 90-09. Specifically, Surveillance Requirements 4.7.6.b and 4.7.6.c will be revised in accordance with the Generic Letter. NRC approval by August 3, 1991, will be requested to permit the use of these revisions for the inspections being conducted during this outage and to ensure appropriate credit for these revisions to establish the frequency for future surveillances.
4. Scheduled date for submitting proposed licensing action and supporting information.
- A. Proposed Change on Autoclosure Interlock - Submitted April 15, 1991
 - B. Proposed Change and Exemption of ILRT - Submitted April 8, 1991
 - C. Approval of Shutdown Cooling for Spent Fuel Pool Cooling - Submitted April 8, 1991
 - D. Proposed Change on Snubber Surveillances - April 30, 1991
5. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None.

REFUELING INFORMATION

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MONTH: March 1991

6. The number of fuel assemblies.

a) In the core. 217

b) In the spent fuel storage pool. 446 (376 Unit 2 Spent
Fuel Assemblies and 70
Unit 1 Spent Fuel
Assemblies

7. Licensed spent fuel storage capacity. 1542 *

Intended change in spent fuel storage capacity. None

* Expanded from 800 to 1542 by License Amendment No. 87 - Facility modification is scheduled to be completed by March 1991.

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

Approximately 2001 (full off load capability)

NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-362
 UNIT NAME: SONGS - 3
 DATE: 4-15-91
 COMPLETED BY: M. M. Farr
 TELEPHONE: (714) 368-9787

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 3
2. Reporting Period: March 1991
3. Licensed Thermal Power (MWt): 3390
4. Nameplate Rating (Gross MWe): 1127
5. Design Electrical Rating (Net MWe): 1080
6. Maximum Dependable Capacity (Gross MWe): 1127
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7)
 Since Last Report, Give Reasons: NA
9. Power Level To Which Restricted, If Any (Net MWe): NA
10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.00</u>	<u>2,160.00</u>	<u>61,344.00</u>
12. Number Of Hours Reactor Was Critical	<u>669.08</u>	<u>2,085.08</u>	<u>46,313.05</u>
13. Reactor Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
14. Hours Generator On-Line	<u>669.08</u>	<u>2,085.08</u>	<u>45,061.57</u>
15. Unit Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
16. Gross Thermal Energy Generated (MWH)	<u>2,227,766.90</u>	<u>6,987,124.12</u>	<u>143,284,765.68</u>
17. Gross Electrical Energy Generated (MWH)	<u>755,640.00</u>	<u>2,388,840.00</u>	<u>48,640,522.50</u>
18. Net Electrical Energy Generated (MWH)	<u>715,158.97</u>	<u>2,269,833.97</u>	<u>45,894,743.30</u>
19. Unit Service Factor	<u>89.93%</u>	<u>96.53%</u>	<u>73.46%</u>
20. Unit Availability Factor	<u>89.93%</u>	<u>96.53%</u>	<u>73.46%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>89.00%</u>	<u>97.30%</u>	<u>69.27%</u>
22. Unit Capacity Factor (Using DER Net)	<u>89.00%</u>	<u>97.30%</u>	<u>69.27%</u>
23. Unit Forced Outage Rate	<u>10.07%</u>	<u>3.47%</u>	<u>7.58%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	<u>NA</u>		
25. If Shutdown At End Of Report Period, Estimated Date of Startup:	<u>NA</u>		
26. Units In Test Status (Prior To Commercial Operation):	Forecast	Achieved	

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

<u>NA</u>	<u>NA</u>
<u>NA</u>	<u>NA</u>
<u>NA</u>	<u>NA</u>

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-361
UNIT NAME: SONGS - 3
DATE: 4-15-91
COMPLETED BY: M. M. Farr
TELEPHONE: (714) 368-9787

MONTH: March 1991

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1	<u>1104.29</u>
2	<u>1099.96</u>
3	<u>1099.33</u>
4	<u>1099.04</u>
5	<u>1097.50</u>
6	<u>1097.83</u>
7	<u>1097.33</u>
8	<u>1079.83</u>
9	<u>1081.88</u>
10	<u>1093.38</u>
11	<u>1088.88</u>
12	<u>1088.46</u>
13	<u>1088.67</u>
14	<u>1091.33</u>
15	<u>414.00</u>
16	<u>0.00</u>

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17	<u>0.00</u>
18	<u>134.46</u>
19	<u>927.67</u>
20	<u>1073.67</u>
21	<u>1094.63</u>
22	<u>1102.50</u>
23	<u>1095.46</u>
24	<u>1100.42</u>
25	<u>1098.29</u>
26	<u>1097.63</u>
27	<u>1097.33</u>
28	<u>1095.79</u>
29	<u>1080.92</u>
30	<u>1079.46</u>
31	<u>1095.79</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: March 1991DOCKET NO: 50-362UNIT NAME: SONGS - 3DATE: 4-15-91COMPLETED BY: M. M. FarrTELEPHONE: (714) 368-9787

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	LER No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
56	910315	F	74.92	A	3	3-91-001	EE BU	CAP ASU	See note below.
<p>Note: The reactor automatically tripped from 100% power on a reactor protection system loss of load (LOL) signal. The LOL signal was generated by a turbine trip, which occurred as a result of a momentary interruption of power to non-1E instrument bus Q-069 during transfer from the non-1E Uninterruptible Power System (UPS) source to the alternate power source. Bus Q-069 is normally supplied power from the UPS inverter. On loss of the UPS, the non-1E UPS static transfer switch automatically transfers bus Q-069 to the alternate 480 VAC B-12 load center source without power interruption. Due to a failure of a capacitor in the constant voltage transformer (CVT) section of the non-1E UPS inverter output, the internal voltage and current momentarily decreased and increased, initiating an automatic transfer. However, since the static switch overcurrent sensors measure only load currents (not internal fault currents), the static switch transfer was not initiated until a decrease in output voltage was detected. With the short circuit occurring directly at the CVT, the internal energy normally stored in the CVT was shunted and unavailable to sustain the loads during the transfer. This resulted in a brief power interruption before the static transfer switch transferred to its alternate source. All capacitors in Unit 2 & 3 inverters were replaced with an upgraded model. In addition, a Temporary Facility Modification (TFM) was installed to split out the Unit 3 Feedwater Control System power supply. In addition, the instrument bus power supplies were reconfigured to ensure loss of non-1E UPS will not cause a trip and will still provide adequate Control Room indication for safe operation. This TFM will be a permanent design change during the Cycle 6 refueling outages for Units 2 & 3. A challenge to a Unit 3 safety valve, which occurred during this event, is discussed further in Attachment A.</p>									

¹F-Forced
S-Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from
Previous Month
5-Reduction of 20%
or greater in the
past 24 hours
6-Other (Explain)

⁴IEEE Std 805-1984

⁵IEEE Std 803A-1983

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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<u>Date</u>	<u>Time</u>	<u>Event</u>
March 1	0001	Unit is in Mode 1 at 100% reactor power. Turbine load at 1150 MWe gross.
March 15	0938	Reactor tripped from 100% power due to momentary loss of power to non-1E UPS. Entered Mode 3.
March 18	0256	Entered Mode 2 following repair of non-1E UPS power source.
	0315	Reactor made critical.
	0650	Entered Mode 1.
	1233	Unit synchronized to grid. Continued reactor power increase to 70% power.
March 19	0105	Reactor at 70% power. Commenced special test of Feedwater Control System.
	0200	Commenced reactor power increase to 95% following completion of special test of Feedwater Control System.
	1230	Reactor at 95% power. Commenced Excore NI calibration.
	1440	Commenced reactor power increase to 100% following completion of Excore NI calibration.
	1755	Reactor at 100% power.
March 20	1645	Commenced reactor power decrease to 95% following closure of High Pressure Governing Valve 3UV-2200G.
March 20	2335	Commenced reactor power increase to 100% following High Pressure Governing Valve 3UV-2200G repairs.
March 21	0400	Reactor at 100% power.
March 31	2400	Unit is in Mode 1 at 100% reactor power. Turbine load at 1140 MWe gross.

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REFUELING INFORMATION

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MONTH: March 1991

1. Scheduled date for next refueling shutdown.

Cycle 6 refueling outage is forecast for January 1992.

2. Scheduled date for restart following refueling.

Restart from Cycle 6 refueling outage is forecast for April 1992.

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes.

What will these be?

All changes listed in Unit 2 Refueling Information, Item 3, are also applicable to Unit 3.

4. Scheduled date for submitting proposed licensing action and supporting information.

Same dates as specified for Unit 2.

5. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Not yet specifically determined for Cycle 6. Under evaluation.

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MONTH: March 1991

6. The number of fuel assemblies.

a) In the core. 217

b) In the spent fuel storage pool. 445 (376 Unit 3 Spent
Fuel Assemblies and 69
Unit 1 Spent Fuel
Assemblies

7. Licensed spent fuel storage capacity. 1542 *

Intended change in spent fuel storage capacity. None

* Expanded from 800 to 1542 by License Amendment No. 77 - Facility modification is scheduled to be completed by September 1991.

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

Approximately 2003 (full off load capability)

ATTACHMENT A

Challenge to Unit 3 Safety Valve

SONGS Units 2 and 3 Monthly Operating Report

DESCRIPTION OF A CHALLENGE TO
UNIT 3 SAFETY VALVE

On March 15, 1991, at 0938, with Unit 3 at 100% power, an automatic reactor trip occurred on a reactor protection system loss of load (LOL) signal. The LOL signal was caused by a turbine trip, which occurred as a result of a momentary loss of power to non-1E instrument bus Q-069.

On the secondary side of the steam generators (SG), control room operators opened the atmospheric dump valves (ADVs) to reduce SG pressure. SG pressure peaked several seconds later at approximately the lift setpoint pressure of the first Main Steam Safety Valve (MSSV) for each SG. One MSSV for SG E-088 was determined to have briefly lifted and properly reseated. There are two main steam lines for Unit 2, each provided with nine safety valves, for a total of 18 safety valves. The first MSSV is set to open at 1100 psia with succeeding valves to open one for each additional 7 psia of pressure.

The reporting of a challenge to a safety valve is required by Section 6.9.1.10 of Appendix A, Technical Specification to Facility Operating License NPF-15 for San Onofre Unit 3. Additional information regarding the trip and the challenge to the safety valve is provided in Licensee Event Report No. 91-001, Docket No. 50-362.