

SEP 28 1978

Docket Nos. 50-361  
and 50-362

Mr. James H. Drake  
Vice President  
Southern California Edison Company  
2244 Walnut Grove Avenue  
P. O. Box 800  
Rosemead, California 91770

Mr. B. W. Gilman  
Senior Vice President - Operations  
San Diego Gas and Electric Company  
101 Ash Street  
P. O. Box 1831  
San Diego, California 92112

Gentlemen:

SUBJECT: ADDENDA TO NRC LETTER OF SEPTEMBER 20, 1978

Enclosed is page 212-1 of our letter of September 20, 1978. This page was inadvertently omitted from the September 20, 1978 letter.

Please contact us if you have any questions about the information requested.

Sincerely,

Original signed by  
~~Robert L. Baer~~  
Robert L. Baer, Chief  
Light Water Reactors Branch No. 2  
Division of Project Management

Enclosure:  
Request for Additional  
Information

ccs w/enclosure:  
See pages 2 and 3

Ap 3  
GD

OFFICE >	LWR #2:LPM	LWR #2:BC				
SURNAME >	HRood:ch <i>HR</i>	RBaer <i>RBaer</i>				
DATE >	9/27/78	9/28/78				

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Mr. James H. Drake  
Mr. B. W. Gilman

SEP 28 1978

cc: Mr. R. W. DeVane, Jr.  
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San Clemente, California 92672

212.0 REACTOR SYSTEMS BRANCH

212.140 (5.2.5) Your response to staff question 212.114 on identified leakage is not satisfactory. It is the staff's position that steam generator primary to secondary leakage is unidentified leakage. Unidentified leakage from the primary system is monitored to identify potential serious degradation of the reactor coolant pressure boundary.

Justify that steam generator primary to secondary leakage at a rate of 1 gpm will be detected in at most 1 hour.

212.141 (5.2.2) Your response to question 212.117 on water-solid overpressurization is not complete. Provide a discussion of the following topics as they pertain to water-solid overpressure events:

- (1) Describe how you have accounted for the flashing of water at the discharge of your water-solid overpressurization relief valve in the sizing of the valve or in your over-pressure analysis.
- (2) Justify that water-solid overpressure transients will not exceed 110 percent of the design pressure of any component in the SDCS.

## Distribution

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LWR #2 File  
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