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SUBJECT: Provides addl info re licensee Proposed Changes Notices 275
 & 280, per request for addl details.

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March 12, 1990

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Docket Nos. 50-361 and 50-362
Amendment Application Nos. 69 and 55 (NPF-10/15-275) and
Amendment Application Nos. 72 and 58 (NPF-10/15-280)
San Onofre Nuclear Generating Station
Units 2 and 3

Reference: 1. SCE to NRC letter, dated 1/8/90, regarding Amendment
Applications 69 and 55, Proposed Change Notice NPF-10/15-275
2. SCE to NRC letter, dated 1/8/90, regarding Amendment
Applications 70 and 56, Proposed Change Notice NPF-10/15-276.
3. SCE to NRC letter, dated 1/8/90, regarding Amendment
Applications 72 and 58, Proposed Change Notice NPF-10/15-280

This letter provides additional information regarding Southern California Edison's (SCE) Proposed Change Notices (PCNs) 275 and 280. This information is submitted in response to a recent telephone request from the NRC Project Manager to provide additional details for these proposed changes. PCNs 275 and 280 are requests to extend the "refueling" interval surveillances from 18+25% months to 24+25% months. The information requested is supplied as an enclosure to this letter.

The Reactor Protective System (RPS) and Engineered Safety Feature Actuation System (ESFAS) evaluation for calibration extension resulted in changes to Technical Specification setpoints. The revised trip setpoints are based on changes to the Safety Analysis Setpoints and changes to the trip setpoint calculations. No changes to the Safety Analysis Limits were made.

Both SCE and Combustion Engineering (C-E) reviewed all Chapter 15 events and the effect of the extended surveillance intervals on these events. The C-E review consisted of a verification of all trips that are credited in all Chapter 15 analyses. This review is documented in the proposed changes (PCN 275 and 280) and summarized in tabular form in the enclosure to this letter.

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March 12, 1990

For each FSAR Chapter 15 Design Basis Event, the table provides the credited RPS trip and the ESFAS functions activated. In addition, those RPS trips and Emergency Feedwater Actuation System functions, for which the Safety Analysis Setpoints were changed, are specifically addressed. These include High Pressurizer Pressure, Low Pressurizer Pressure, High Containment Pressure, Low Steam Generator Water Level, and High Steam Generator Differential Pressure. The effect of those changes are summarized for each of the Chapter 15 events. Changes to the Containment High pressure setpoint which affect ESFAS functions are addressed in Attachment H to References 1 and 3, but are not included in this table. This information was used in the development of the 10 CFR 50.92 Safety Analysis performed for the proposed changes.

In addition, this letter submits Revision 0, of Report No. M-89047, "Instrument Drift Study." This Instrument Drift Study was used in support of the instrumentation proposed changes for Reactor Protective System (PCN 275), Remote Shutdown System (PCN 276), and Engineered Safety Feature Actuation System (PCN 280).

If you have any questions regarding this additional information, please call me.

Very truly yours,

Enclosures

cc: J.B. Martin, Regional Administrator, NRC Region V
C. Caldwell, NRC Senior Resident Inspector, San Onofre Units 1, 2 and 3