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ACCESSION NBR:9204170195 DOC.DATE: 92/04/13 NOTARIZED: NO DOCKET #
 FACIL:50-361 San Onofre Nuclear Station, Unit 2, Southern Californ 05000361
 AUTH.NAME AUTHOR AFFILIATION
 KRIEGER,R.W. Southern California Edison Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-005-00:on 920312,determined that MSIV bypass valves not included in ASME,Section XI inservice testing program. Caused by failure to implement change to TS 3/4.6.3.Valves tested satisfactorily.W/920413 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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INTERNAL:	ACNW		2	2		ACRS		2	2
	AEOD/DOA		1	1		AEOD/DSP/TPAB		1	1
	AEOD/ROAB/DSP		2	2		NRR/DET/EMEB 7E		1	1
	NRR/DLPQ/LHFB10		1	1		NRR/DLPQ/LPEB10		1	1
	NRR/DOEA/OEAB		1	1		NRR/DREP/PRPB11		2	2
	NRR/DST/SELB 8D		1	1		NRR/DST/SICB8H3		1	1
	NRR/DST/SRLB8D1		1	1		NRR/DST/SRXB 8E		1	1
	REG FILE 02		1	1		RES/DSIR/EIB		1	1
	RGNS FILE 01		1	1					
EXTERNAL:	EG&G BRYCE, J.H		3	3		L ST LOBBY WARD		1	1
	NRC PDR		1	1		NSIC MURPHY, G.A		1	1
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Southern California Edison Company

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STATION MANAGER

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April 13, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 92-005
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(d), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the failure to inservice test the main steam isolation bypass valves. Since this occurrence involves similar systems, cause, and corrective actions applicable to Units 2 and 3, a single report for Unit 2 is being submitted in accordance with NUREG-1022. Neither the health nor the safety of plant personnel or the public was affected by this occurrence.

If you require any additional information, please so advise.

Sincerely,

M. P. Stoltz for
R. W. Krieger

Enclosure: LER No. 92-005

cc: C. W. Caldwell (USNRC Senior Resident Inspector, Units 1, 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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LICENSEE EVENT REPORT (LER)																	
Facility Name (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2												Docket Number (2) 0 5 0 0 0 3 6 1				Page (3) 1 of 0 4	
Title (4) MAIN STEAM ISOLATION BYPASS VALVES NOT INSERVICE TESTED IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS																	
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
Month	Day	Year	Year	/// Sequential Number	/// Revision Number	Month	Day	Year	Facility Names				Docket Number(s)				
0 3	1 2	9 2	9 2	0 0 5	0 0	0 4	1 3	9 2	SONGS, UNIT 3				0 5 0 0 0 3 6 2				
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)														
POWER LEVEL (10) 1 0 0			20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		Other (Specify in Abstract below and in text)						
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)								
////////////////////			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)										
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)										
////////////////////			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)										
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)										
LICENSEE CONTACT FOR THIS LER (12)																	
Name R. W. Krieger, Station Manager												TELEPHONE NUMBER AREA CODE 7 1 4 3 6 8 - 6 2 5 5					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS								
SUPPLEMENTAL REPORT EXPECTED (14)										Expected Submission Date (15)		Month		Day		Year	
<input type="checkbox"/> Yes (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO																	
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																	

At 0930 on 3/12/92, with Unit 2 in Mode 1 and Unit 3 in Mode 5, it was determined that the main steam isolation valve (MSIV) bypass valves (two valves each for both Units 2 and 3) had not been included in the ASME, Section XI inservice testing (IST) program as required by Technical Specification (TS) 3/4.6.3, "Containment Isolation Valves." Testing was being performed semiannually in accordance with TS 3/4.3.2, "Engineered Safety Feature Actuation System Instrumentation," rather than quarterly as required by the ASME Code. The capability of the valves to function properly was therefore regularly demonstrated but at a frequency which did not satisfy the surveillance requirements of TS 3/4.6.3.

The bypass valves were excluded from the IST program as a result of not fully implementing a change to TS 3/4.6.3 in May 1986, which added the testing requirement for these and other valves. Due to an apparent oversight by individuals responsible for implementation of changes to the TSs, the organization responsible for oversight of the IST program was not adequately advised of the changes such that program revisions could be incorporated. The MSIV bypass valves have since been tested satisfactorily per the applicable ASME, Section XI criteria, and have been added to the IST program.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

SAN ONOFRE NUCLEAR GENERATION STATION	DOCKET NUMBER	LER NUMBER	PAGE
UNIT 2	05000361	92-005-00	2 of 4

Plant: San Onofre Nuclear Generating Station
 Units: Two & Three
 Reactor Vendor: Combustion Engineering
 Event Date: 3-12-92
 Time: 0930

A. CONDITIONS AT TIME OF THE EVENT:

Unit 2: Mode 1, 100 % Power Operation
 Unit 3: Mode 5, Cold Shutdown

B. BACKGROUND INFORMATION:

1. Main Steam Isolation

A Main Steam Isolation Signal (MSIS) [JE] is generated by a low steam generator (SG) [SG] pressure signal, which is indicative of a main steam line break. The MSIS then initiates closure of certain main steam and main feedwater supply isolation valves [ISV] including the main steam isolation valves (MSIV) and MSIV bypass valves (HV-8202 and HV-8203) [FCV]. The MSIV bypass valves are 4-inch, pneumatically operated valves. Their primary function is to provide a means to warm-up the main steam piping downstream of the MSIVs during plant startup and to equalize pressure across the MSIVs prior to opening.

2. Containment Isolation System

The containment isolation system (CIS) [JM] consists of piping, valves, instrumentation and controls required to isolate the containment following: 1) a Loss of Coolant Accident (LOCA) to minimize release of radioactive materials to the atmosphere, and 2) a main steam line break to prevent excessive cooldown of the reactor coolant system (RCS) [AB]. Closure of the MSIV bypass valves is necessary to isolate the SGs if required to prevent excessive cooldown of the RCS; they are therefore considered components of the CIS.

3. Technical Specification Requirements

TS 3/4.6.3, "Containment Isolation Valves," Surveillance 4.6.3.5 requires that the isolation valves specified in Section D of Table 3.6-1 be demonstrated operable as required by Specification 4.0.5. Section D contains a listing of valves which operate automatically on an Engineering Safety Feature Actuation Signal (ESFAS) signal to mitigate the consequences of a potential design basis accident. The MSIV bypass valves are among the valves listed in Table 3.6-1, Section D.

Technical Specification (TS) 3/4.3.2, "Engineered Safety Feature Actuation System Instrumentation," Surveillance 4.3.2.1 requires, in

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

SAN ONOFRE NUCLEAR GENERATION STATION	DOCKET NUMBER	LER NUMBER	PAGE
UNIT 2	05000361	92-005-00	3 of 4

part, that the MSIS instrumentation channels be demonstrated operable. As part of this test, closure of the MSIV bypass valves is verified to occur upon receipt of a MSIS.

TS Surveillance Requirement 4.0.5 requires that inservice inspection and testing of ASME Code Class 1, 2, and 3 components be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g).

C. DESCRIPTION OF THE EVENT:

1. Event:

At 0930 on 3/12/92, with Unit 2 in Mode 1 and Unit 3 in Mode 5, it was determined that the MSIV bypass valves (two valves each for both Units 2 and 3) had not been included in the ASME, Section XI inservice testing (IST) program and tested as required by TS Surveillance 4.6.3.5. Testing was being performed semiannually in accordance with TS 3/4.3.2 rather than quarterly as required by the ASME Code. The capability of the valves to function properly was therefore regularly demonstrated but at a frequency which did not satisfy the surveillance requirements of TS 3/4.6.3.

2. Inoperable Structures, Systems or Components that Contributed to the Event:

None.

3. Sequence of Events:

Not applicable.

4. Method of Discovery:

In response to an inquiry associated with post-maintenance testing of the MSIV bypass valves, it was determined that the MSIV bypass valves were not being tested in accordance with TS Surveillance 4.6.3.5 requirements.

5. Personnel Actions and Analysis of Actions:

Not applicable.

6. Safety System Responses:

Not applicable.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

SAN ONOFRE NUCLEAR GENERATION STATION UNIT 2	DOCKET NUMBER 05000361	LER NUMBER 92-005-00	PAGE 4 of 4
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D. CAUSE OF THE EVENT:

The MSIV bypass valves were not included in the IST program as a result of not fully implementing the requirements of TS amendment Nos. 46 and 35 for Units 2 and 3, which were issued in May of 1986. These amendments revised portions of TS 3/4.6.3, including adding the requirement to test the valves in Table 3.6-1 in accordance with the ASME, Section XI IST program specified in TS 4.0.5. Due to an apparent oversight by individuals responsible for implementation of changes to the TSs, the organization responsible for oversight of the IST program was not adequately advised of the change such that program revisions could be incorporated.

E. CORRECTIVE ACTIONS:

1. Corrective Actions Taken:

- a. TS Table 3.6-1 valves have been reviewed to ensure that they are properly included in the IST program.
- b. The Unit 2 and 3 MSIV bypass valves were satisfactorily tested.
- c. The MSIV bypass valves have been added to the IST program.

2. Planned Corrective Actions:

This event will be reviewed with the personnel responsible for ensuring that changes to the TSs are properly reviewed for implementation.

F. SAFETY SIGNIFICANCE OF THE EVENT:

There was no safety significance to this event since subsequent inservice testing indicated satisfactory valve performance. In addition, the semiannual testing has demonstrated that the MSIV bypass valves have been capable of performing their required safety function.

G. ADDITIONAL INFORMATION:

1. Component Failure Information:

Not applicable.

2. Previous LERs for Similar Events:

None.