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 AUTH. NAME AUTHOR AFFILIATION
 MORGAN, H.E. Southern California Edison Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 89-010-00: on 890520, main steam safety valve flow capacity apparently less than nameplate rating.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P. O. BOX 128

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June 30, 1989

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 89-010
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.73(d), this submittal provides a voluntary Licensee Event Report (LER) for an occurrence involving Unit 2 and Unit 3 Main Steam Safety Valve capacity. Since this occurrence involved similar systems, cause and corrective actions, a single report is being submitted in accordance with NUREG-1022. Neither the health and safety of plant personnel or the public was affected by this occurrence.

If you require any additional information, please so advise.

Sincerely,

H E Morgan

Enclosure: LER No. 89-010

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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LICENSEE EVENT REPORT (LER)																				
Facility Name (1)										Docket Number (2)						Page (3)				
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2										0 5 0 0 0 3 6 1						1 of 0 1				
Title (4)																				
MAIN STEAM SAFETY VALVE FLOW CAPACITY APPARENTLY LESS THAN NAMEPLATE RATING																				
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
Month	Day	Year	Year	///	Sequential	///	Revision	Month	Day	Year	Facility Names				Docket Number(s)					
				///	Number	///	Number													
0 5	2 0	8 9	8 9	---	0 1 0	---	0 0	0 6	3 0	8 9	SONGS, UNIT 3				0 5 0 0 0 3 6 2					
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)																	
POWER LEVEL (10)			20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)		
0 0 0			20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)		
			20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					X Other (Specify in		
			20.405(a)(1)(iii)					50.73(a)(2)(i)					50.73(a)(2)(viii)(A)					Abstract below and		
			20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)					in text)		
			20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(x)					VOLUNTARY		
LICENSEE CONTACT FOR THIS LER (12)																				
Name										TELEPHONE NUMBER										
H. E. Morgan, Station Manager										AREA CODE										
										7 1 4		3 6 8 - 6 2 4 1								
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																				
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	//////	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	//////									
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B	S	B		R V	C 7 1 0	Y					//////									
SUPPLEMENTAL REPORT EXPECTED (14)										Expected Submission Date (15)		Month	Day	Year						
Yes (If yes, complete EXPECTED SUBMISSION DATE)										X NO										
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																				

On 5/20/89, with Unit 2 in Mode 5, Cold Shutdown and Unit 3 at 100% power, during an ongoing evaluation of Information Notice 86-05 (IN), it was concluded that the Main Steam Safety Valve (MSSV)[RV] flow capacities are most likely less than nameplate rating (approximately 233 pounds mass per second at 103% of safety valve set pressure). This conclusion was based upon best available findings of the Westinghouse Owners Group (WOG) Subcommittee on MSSVs. The WOG is evaluating reduced MSSV capacities reported at other facilities and in the IN. Conservative evaluation techniques (e.g. a MSSV analytical model which estimates full nameplate flow at approximately 108% of set pressure) indicate that the MSSV capacities are currently sufficient to meet design bases and would not alter the results of safety analyses. In addition, these conclusions are supported by actual performance data taken following a Unit 2 loss of heat removal event (LER 86-022). The MSSVs are manufactured by Crosby Valve and Gage Co. (Model No. 6R10 HA75FN). This voluntary LER is being submitted to provide notification of these findings.

The apparent reduction of MSSV capacity is believed to be a result of manufacturer error in adjusting the blowdown and accumulation characteristics of the valves as reported in the IN. As corrective action, the MSSVs will be readjusted at the next Unit 2 and Unit 3 refueling outages.

The MSSVs satisfy Technical Specification (TS) 3/4.7.1.1, Safety Valves, which requires the MSSVs to be operable with specific set pressures. Also, they satisfy the Bases for this TS by limiting steam pressure to 110% of design during certain severe transients. However, the evaluation has revealed that the MSSVs' total flow rate stated in the TS Bases and in FSAR Appendix 5.2A, Over Pressure Protection, is a preliminary value used prior to manufacture rather than the flow needed to limit steam pressure. Correction of these items and supporting documentation will be provided by a TS Amendment Request and the next FSAR annual update.