



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket Nos: 50-361, (362)

April 22, 1977

Southern California Edison Company  
ATTN: Mr. Jack B. Moore, Vice President  
2244 Walnut Grove Avenue  
P. O. Box 800  
Rosemead, California 91770  
Gentlemen:

SUBJECT: STANDARD FORMAT FOR METEOROLOGICAL DATA ON MAGNETIC TAPE

Regulatory Guide 1.70, Revision 2, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants" (September 1975), recommends that, if possible, hour-by-hour meteorological data should be provided on magnetic tape. The draft Environmental Standard Review Plan for meteorology (included in NUREG-0158, Part 1, January 1977) provides some guidance on the types of magnetic tapes that are acceptable and identifies limitations on block size and density. We have also developed a standardized format (Enclosure 1) for providing hour-by-hour meteorological data on magnetic tapes that would facilitate our review of atmospheric diffusion characteristics and cooling system impacts. Enclosure 2 provides a sample tape dump using this format. We would prefer receiving meteorological data magnetic tapes in the standard format; however, we will accept data on magnetic tape in any reasonable format, if the format is completely described (per guidance in NUREG-0158, Part 1) and a sample tape dump is provided. Questions on this standard format should be referred to William Snell, Hydrology-Meteorology Branch, Division of Site Safety and Environmental Analysis, phone number 301-492-7384.

This request for generic information was approved by GAO under blanket clearance number B-180225 (R0072). This clearance expires July 31, 1977.

Sincerely,

A handwritten signature in dark ink, appearing to read "D. Vassallo".

Dominic B. Vassallo, Assistant Director  
for Light Water Reactors  
Division of Project Management

Enclosures:  
As Stated

MA4  
GD

Enclosure 1

PROPOSED FORMAT FOR HOURLY METEOROLOGICAL  
DATA TO BE PLACED ON MAGNETIC TAPE

Use: 9 track tape (7 will be acceptable)

Standard Label which would include

Record Length = 160

Block Size (3200 - fixed block size)

Density (1600 BPI - 800 will be accepted)

Do Not Use: Magnetic tapes with unformatted or spanned records.

At the beginning of each tape, use the first five (5) records (which is the equivalent of ten cards) to give a tape description. Include plant name, and location (latitude, longitude) dates of data, information explaining data contained in the "other" fields if they are used, height of measurements, and any additional information pertinent to identification of the tape. Make sure all five records are included, even if some are blank. Format for the first five records will be 160A1. Meteorological data format is (I6, I2, I3, I4, 25F5.1, F5.2, 3F5.1). Decimal points should not be included when copying data onto the tape.

All data should be given to a tenth of a unit except solar radiation which should be given to a hundredth of a unit.

This does not necessarily indicate the accuracy of the data.

(e.g. wind direction is usually given to the nearest degree but record it with a zero in the tenth's place. That is 275 degrees would be 275.0 degrees and placed on the tape as 2750.)

All nines in any field indicates a lost record (99999). All sevens in a wind direction field indicates calm (77777).

If only two levels of data, use the upper & lower levels. If only one level of data, use the upper level.

Enclosure 1

MAGNETIC TAPE  
METEOROLOGICAL DATA

LOCATION:

DATE OF DATA RECORD:

<u>I6</u>	Identifier (can be anything)	
<u>I2</u>	Year	
<u>I3</u>	Julian Day	
<u>I4</u>	Hour (on 24 hr clock)	
		<u>ACCURACY</u>
<u>F5.1</u>	Upper Measurements: Level = meters	
<u>F5.1</u>	Wind Direction (degrees)	_____
<u>F5.1</u>	Wind Speed (meter/sec)	_____
<u>F5.1</u>	Sigma Theta (degrees)	_____
<u>F5.1</u>	Ambient Temperature (°C)	_____
<u>F5.1</u>	Moisture: _____	_____
<u>F5.1</u>	Other: _____	_____
<u>F5.1</u>	Intermediate Measurements: Level = meters	
<u>F5.1</u>	Wind Direction (degrees)	_____
<u>F5.1</u>	Wind Speed (meters/sec)	_____
<u>F5.1</u>	Sigma Theta (degrees)	_____
<u>F5.1</u>	Ambient Temperature (°C)	_____
<u>F5.1</u>	Moisture: _____	_____
<u>F5.1</u>	Other: _____	_____

Enclosure 1

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<u>F5.1</u>	Lower Measurements: Level = meters	
<u>F5.1</u>	Wind Direction (degrees)	_____
<u>F5.1</u>	Wind Speed (meters/sec)	_____
<u>F5.1</u>	Sigma Theta (degrees)	_____
<u>F5.1</u>	Ambient Temperature (°C)	_____
<u>F5.1</u>	Moisture: _____	_____
<u>F5.1</u>	Other: _____	_____
<u>F5.1</u>	Temp Diff (Upper-Lower) (°C/100 meters)	_____
<u>F5.1</u>	Temp Diff (Upper-Intermediate) (°C/100 meters)	_____
<u>F5.1</u>	Temp Diff (Intermediate-Lower) (°C/100 meters)	_____
<u>F5.1</u>	Precipitation (mm)	_____
<u>F5.2</u>	Solar Radiation (cal/cm <sup>2</sup> /min)	_____
<u>F5.1</u>	Visibility (km)	_____
<u>F5.1</u>	Other: _____	_____
<u>F5.1</u>	Other: _____	_____

# Enclosure 2

NUCLEAR POWER FACILITY - METEOROLOGICAL DATA TAPE  
 974-JUNE 1975 TAPE CREATED MARCH 1977 LOCATION: LAT=38 DEG 00 MIN 00 SEC N LG  
 NG=0 DEG 00 MIN 00 SEC W WIND DIRECTION: 150 FT AND 33 FT DATA TO NEAREST 1.0 DEG  
 REE WIND SPEED: 150 FT AND 33 FT DATA TO NEAREST 0.1 METER/SEC  
 DELTA T (150-33FT): DATA TO NEAREST 0.1 DEGREE C/100 METERS TEMPERATURE: 150 AND  
 33 FT DATA TO NEAREST 0.1 DEGREE C THIS IS A COPY OF THE ORIGINAL TPA TAPE  
 WITH THE FORMAT CHANGED AND THE UNITS CHANGED AS FOLLOWS: DEGREES F TO DEGREES C 1 MPH TO METER  
 3/SEC 1 FT TO METERS OTHER FIELDS: WIND DIRECTION PERSISTENCE (%)

74182 100 457 1900 25 225 73 100 1700 26 221	74182 200 457 1800 0 23 212 1000
100 1700 4 201 0 39 74182 300 457 1900 29 215 1000	74182 400 457 195 0 960
1890 35 211 1000 74182 500 457 1900 37 205 100 1700 9 195 0	
44 100 1600 9 191 0 41 74182 600 457 1950 43 197 1000	100 1700 9 194
74182 800 457 2100 201 0 9 860 100 1800 9 187 0 20 980	74182 700 457 2050 38 194
100 1800 4 201 0 9 860 74182 900 457 2140 31 218 860	100 2000 13 220 0 830
2140 -6 100 2000 22 249 0 -11 741821000 457 2170 38 245 100 2300 31 266	
741821100 457 2420 262 0 -11 750 100 2300 36 271 0 -12 590	741821200 457 2340 53 266
100 2100 36 280 0 -12 660 741821300 457 2440 280 0 -12 660	100 2100 27 285 0 590
2210 -12 100 1900 22 291 0 -11 741821500 457 1870 44 287 590	
741821600 457 1930 292 0 -9 960 100 1700 22 292 45 292	REC 2, LENGTH 3200
100 1800 18 292 0 -5 980 741821800 457 1930 42 291 960	
1740 -3 100 1600 4 276 0 2 741822000 457 1800 29 276 0 2	
741822100 457 1770 260 0 16 930 100 1700 4 248 1000	741822200 457 1700 35 248 1000
100 1500 4 239 0 25 980 741822300 457 1700 239 0 25 980	100 1700 33 248 1000
1760 22 100 1600 4 228 0 20 1000 74183 100 457 1820 228 0 20 1000	
74183 200 457 1790 221 0 26 940 100 1600 4 215 1000	74183 300 457 1820 228 0 20 1000
100 1500 4 214 0 20 980 74183 400 457 1830 214 0 20 980	100 1500 25 25 1000
1810 17 100 1400 4 208 0 25 940 74183 600 457 1780 208 0 25 940	
74183 700 457 1810 205 0 23 780 74183 800 457 1780 205 0 23 780	
100 1600 9 216 1000	

The first data record gives the following:

Year: 1974  
 Day: 182  
 Time: 0100

Height: 45.7 meters  
 Wind Direction: 198 degrees  
 Wind Speed: 2.9 m/sec  
 Ambient Temperature: 22.5°C  
 Wind Direction Persistence: 73%

Height: 10.0 meters  
 Wind Direction: 170 degrees  
 Wind Speed: 0.4 m/sec  
 Ambient Temperature: 21.8°C  
 Wind Direction Persistence: 0%

Temperature Difference (Upper-Lower): 1.9°C/100 meters

Distribution

Docket File

NRC PDR

Local PDR

LWR #2 File

RSBoyd

DEVassallo

LCrocker

HSmith

FJWilliams

VStello

RHeineman

JMcGough

TIppolito

RTedesco

JMiller

IE (3)

HRood

JLee

ACRS (16)

JRBuchanan, NSIC

TBAbernathy, TIC

APR 11 1977

Docket Nos: 50-361

and 50-362

Southern California Edison Company

ATTN: Mr. Jack B. Moore

Vice President

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P. O. Box 800

Rosemead, California 91770

San Diego Gas & Electric Company

ATTN: Mr. Jack E. Thomas

Vice President - Electric

101 Ash Street

P. O. Box 1831

San Diego, California 92112

Gentlemen:

SUBJECT: INSTRUMENT TRIP SETPOINT VALUES

(San Onofre Nuclear Generating Station, Units 2 and 3)

Our review of facility operating experience indicates the need for additional information regarding the proper selection of instrumentation trip setpoint values. This conclusion is supported by the large number of Licensing Event Reports (LERs) received by the Commission related to instrument setpoint drift beyond the limits permitted by facility technical specifications.

We have structured the Standard Technical Specifications (STS) to minimize the need for licensees to submit LERs related to instrument drift provided that allowances for drift have been properly accounted for in the analyses supporting the selection of the trip setpoint values.

In view of the above, we require explicit information concerning each Reactor Protection System (RPS) and Engineered Safeguards Features (ESF) trip setpoint value as part of the documentation to be provided for review of your facility.

Since your FSAR does not contain sufficient information necessary for our evaluation, you should provide the following values for each RPS and ESF instrumentation channel:

- (a) The technical specification trip setpoint value,
- (b) The technical specification allowable value (i.e., the technical specification trip setpoint plus the instrument drift assumed in the accident analysis),
- (c) The instrument drift assumed to occur during the interval between technical specification surveillance tests,

OFFICE >					
SURNAME >					
DATE >					

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San Diego Gas and Electric Company

- (d) The components of the cumulative instrument bias (e.g., instrument calibration error, instrument drift, instrument error, etc.), and
- (e) The minimum margin between the technical specification trip setpoint and the trip value assumed in the accident analysis.

You should advise us within 30 days of receipt of this letter, your proposed schedule for submittal of this information.

Should you desire further information regarding this subject, please contact us.

Sincerely,

Original Signed by

Karl Kniel, Chief  
Light Water Reactors Branch No. 2  
Division of Project Management

cc: See Page 3

OFFICE ➤	DPM:LWR #2	DPM:LWR #2	DPM:LWR #2			
SURNAME ➤	JLee:mt	HRood	KKniel			
DATE ➤	3/28/77	3/ /77	3/ /77			

Southern California Edison Company  
San Diego Gas and Electric Company

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APR 11 1977

cc: Rollin E. Woodbury, General Counsel  
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SURNAME ➤						
DATE ➤						



MAR 30 1977

Docket Nos. 50-361  
and 50-362

~~Distribution~~

Docket File	FWilliams	GChipman	TAbernathy,
NRC PDR	HSmith	JMiller	TIC
Local PDR	LCrocker	IE (3)	
OELD	RHeineman	KGoller	
KKniel	HDenton	VStello	
HRood	RVollmer	JYore, ASLBP	
JLee	RTedesco	ARosenthal, ASAB	
RSEoyd	WGammill	ACRS (16)	
DEVassallo	DBunch	JBuchanan, NSIC	

Southern California Edison Company

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ATTN: Mr. Jack E. Thomas

Vice President - Electric

101 Ash Street

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Gentlemen:

SUBJECT: FUEL HANDLING ACCIDENT

(San Onofre Nuclear Generating Station, Units 2 and 3)

We are in the process of evaluating a refueling accident inside the containment building since it may not have been adequately considered in the licensing review.

Based on our preliminary review, potential site boundary radiation exposures due to such an accident at your facility would be within the exposure guidelines of 10 CFR Part 100 even assuming no isolation of containment.

In order to confirm these results and determine if the acceptance criteria of Standard Review Plan 15.7.4 are met, and to document the factors involved in the evaluation, we request that you provide a detailed evaluation of the potential consequences of such an accident at your facility in your PSAR. Your analysis should utilize assumptions comparable to those given in Regulatory Guide 1.25, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors," assuming the worst single failure. It should consider, in a conservative manner, any mixing in the containment atmosphere which would delay release of material, any filtration of effluent which would reduce releases, and any automatic isolation of the containment which would limit releases. Your analysis should utilize parameters (e.g., maximum allowable valve closure times) as limited by the proposed technical specifications. Clearly indicate any credit taken in the analyses for nonsafety grade equipment and provide appropriate justification for this credit. Detailed questions which should be addressed in your response are enclosed.

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SURNAME ➤						
DATE ➤						

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You should compare proposed technical specifications (e.g., valve closure times, filter-testing) with results of the above analyses to show that parameters important in the evaluation are maintained at levels which will assure that conservatively calculated offsite consequences are well within the exposure guidelines of 10 CFR Part 100 over the plant lifetime.

Submit your analysis with your responses to Q-1 (not currently scheduled, but we anticipate Q-1 response to be scheduled for mid-summer, 1977).

This request for generic information was approved by GAO under a blanket clearance number B-180225 (R0072). This clearance expires July 31, 1977.

Original signed by  
K. Kniel

Karl Kniel, Chief  
Light Water Reactors Branch No. 2  
Division of Project Management  
Office of Nuclear Reactor Regulation

Enclosure:  
Information Needed to  
Evaluate Containment  
Refueling Accident

cc w/enclosures:  
See page 3

OFFICE >	DPM:LWR #2	DPM:LWR #2	DPM:LWR #2			
SURNAME >	JLee:mt	HRood	KKniel			
DATE >	3/ /77	3/ /77	3/ /77			

Southern California Edison Company  
San Diego Gas and Electric Company - 3 -

MAR 30 1977

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OFFICE➤						
SURNAME➤						
DATE➤						

INFORMATION NEEDED TO EVALUATE CONTAINMENT ISOLATION CAPABILITY  
DURING REFUELING ACCIDENT

- 1) Describe all instrumentation which would detect a fuel-handling accident (FHA) inside containment. Your response should include the following information:
  - a) instrumentation function, e.g., close containment isolation valves;
  - b) type of instruments and setpoints, e.g., mr/hr, and normal background reading;
  - c) safety class, redundancy, power sources, and technical specification requirements;
  - d) a description of instrument response following a FHA taking into account instrument location;
  - e) response time for the instrument to signal containment isolation after the FHA.
- 2) Describe the response of the containment isolation and ventilation valves following the FHA. Include valve closure times including expected valve closure time as well as technical specification requirements.
- 3) Provide the transit time from the point where a monitor can respond to a release from the FHA to the inboard isolation valve based on the maximum air velocity (peak centerline velocity) at maximum exhaust flow. Also include the transit time based on average velocity and normally expected air flows. Conservatively assume that the FHA is a puff release from the pool at a point closest to an exhaust grill.
- 4) Provide drawings of the containment which clearly show the location of the radiation monitors relative to the ventilation exhaust system including all exhaust inlets, filters, dampers, and duct arrangement up to the outboard isolation valves.

5) If the summation of the instrument response time (question 1.e) and valve dampers closure time (question 2) is greater than the gas transit time (question 3), provide an analysis as to the volume and amount of radioactive exhaust air which could be released. Your response should include the following:

- a) duct sizes;
- b) maximum (peak) air velocity;
- c) average air velocity;
- d) containment isolation valve closure characteristics;
- e) exhaust system flow rates;
- f) methodology used to calculate gas transit times from the pool surface to the exhaust system;
- g) air velocity profiles over the pool surface. You should consider the effects of pool water temperature on air flow trajectories.

6) Describe any charcoal filters which would mitigate the consequence of the FHA. If so, include the following information: type (e.g., kidney), redundancy, power sources, safety grade, technical specification requirements.

In responding to the above, reference can be made to specific pages or figures in the Safety Analysis Report.