



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 9, 2013

Mr. Timothy S. Rausch
Senior Vice President and Chief Nuclear Officer
PPL Susquehanna, LLC
769 Salem Boulevard
Berwick, PA 18603-0467

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2 – REQUEST FOR
ADDITIONAL INFORMATION REGARDING RELIEF REQUEST 3RR-20 TO
THE THIRD 10-YEAR INSERVICE INSPECTION PROGRAM
(TAC NO. MF1756)

Dear Mr. Rausch:

By letter dated May 6, 2013,¹ PPL Susquehanna, LLC (PPL) submitted for Nuclear Regulatory Commission (NRC) staff review and approval, Inservice Inspection Program Relief Request 3RR-20, which requests relief from the ASME Section XI, Code Category B-G-1, Item B6.190 requirement to perform a visual examination of 100% of the 'A' Reactor Recirculation Pump Flange for the third 10-year inspection interval. To complete its review, the NRC staff requests responses to the enclosed questions.

The draft questions were sent to Mr. Jason Jennings, of your staff, to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On May 8, 2013, Mr. Jennings, agreed that you would provide a response by May 31, 2013.

If you have any questions regarding this matter, please contact me at 301-415-4090.

Sincerely,

A handwritten signature in black ink, reading "Jeffrey A. Whited", is positioned above the typed name.

Jeffrey A. Whited, Project Manager
Plant Licensing Branch 1-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-388

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

¹ Agencywide Documents Access and Management System Accession No. ML13127A199.

REQUEST FOR ADDITIONAL INFORMATION

OFFICE OF NUCLEAR REACTOR REGULATION

RELIEF REQUEST 3RR-20

PPL SUSQUEHANNA, LLC

ALLEGHANY ELECTRIC COOPERATION, INC.

SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2

DOCKET NUMBER 50-388

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- RAI 1 Please describe the Susquehanna operational experience with any possible leakage from the Reactor Recirculation Pump Flange mechanical joint, which might be indicative of degradation of the joint materials.
- RAI 2 The submittal states that, after discovery of the missed VT-1 of the 'A' Reactor Recirculation Pump Flange, a VT-1 examination was performed of the assembled joint which achieved 98% of the required coverage. Provide a sketch or some other means of detailing the limitations of the VT-1 examination performed on the 'A' Reactor Recirculation Pump Flange.
- RAI 3 On page 2 of the submittal, the licensee states, in part, that:

PPL will also perform [a] VT-2 examination on the pump prior to startup of Unit 2.

Please describe the plant conditions and hold time applicable to the performance of this VT-2 examination. If the pressure associated with the VT-2 examination will be different than that required by IWB-5221, please provide justification.

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/ra/

Jeffrey A. Whited, Project Manager
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*via e-mail dated

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