

NRR-PMDAPEm Resource

From: Purnell, Blake
Sent: Wednesday, April 24, 2013 3:14 PM
To: Ken.Nicely@exeloncorp.com
Cc: Mozafari, Brenda; Bowen, Jeremy; Mitchel.Mathews@exeloncorp.com
Subject: Quad Cities Relief Request Nos. I5R-01, I5R-02, and I5R-06 - Request for Additional Information

OFFICE OF NUCLEAR REACTOR REGULATION
REQUEST FOR ADDITIONAL INFORMATION
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
EXELON GENERATION COMPANY, LLC
RELIEF REQUEST NOS. I5R-01, I5R-02, and I5R-06
DOCKET NUMBERS 50-254 and 50-265
TAC Nos. ME9668, ME9669, ME9670, ME9671, ME9676, and ME9677

By letter to the U.S. Nuclear Regulatory Commission (NRC) dated September 28, 2012 (Agencywide Documents Access and management System (ADAMS) Accession No. ML12275A070), as supplemented by letter dated November 28, 2012 (ADAMS Accession No. ML12333A262), Exelon Generation Company, LLC (the licensee) submitted several proposed relief requests for the Quad Cities Nuclear Power Station, Units 1 and 2 (QCNPS). The NRC staff determined that the following additional information is necessary to complete the review of relief requests I5R-01, I5R-02, and I5R-06. To support the schedule for approval the response to this request is expected by May 24, 2013.

Based on the staff's review of relief request I5R-01:

1. Discuss in the relief request why the proposed alternative provides reasonable assurance of structural integrity or leak tightness of the subject components.
2. Discuss in the relief request how the specified requirements of this section would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.
3. Provide a technical basis as to why the ultrasonic testing is not required.

Based on the staff's review of relief request I5R-02:

1. Do the augmented inspection programs for intergranular stress corrosion cracking Category B-G (Generic Letter 88-01), service water integrity (Generic Letter 89-13), flow accelerated corrosion (Generic Letter 89-08), and high-energy line break (NRC Branch Technical Position MEB 3-1), as described in the initial QCNPS risk-informed ISI submittal dated November 30, 2000 (ADAMS Accession No. ML003776493), remain unaffected by the risk-informed ISI program developed for the fifth interval?
2. Are the inspection locations in the QCNPS risk-informed ISI programs that have been developed for the fifth 10-year interval the same locations as those in the fourth interval risk-informed ISI programs approved in the NRC staff's January 28, 2004, safety evaluation? If not, please summarize the changes to the program and what caused those changes.
3. If there are changes in the inspection locations for the QCNPS fifth 10-year interval risk-informed ISI programs please provide information for the fifth interval program regarding: examinations, system, components, degradation mechanisms, class, etc. The information expected should be similar to what

is in Tables 2, 3, 4, 5 and 6 of the original submittal of the risk-informed ISI program for the QCNPS third 10-year inservice inspection interval dated November 30, 2000.

Based on the staff's review of relief request I5R-06:

1. Confirm whether NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking," will be used for the inspection of feedwater sparger tee welds and feedwater sparger piping brackets.
2. Section 4.1 item 5 of the BWRVIP-100-A report, "Updated Assessment of the Fracture Toughness of Irradiated Stainless Steel for BWR Core Shrouds," states that fracture toughness values of stainless steel materials that are exposed to a neutron fluence value greater than 1×10^{21} n/cm² ($E > 1$ MeV) are lower than those used in Appendix C of the BWRVIP-76 report, "BWR Core Shroud Inspection and Flaw Evaluation Guidelines." Identify whether the core shroud welds and base materials will be exposed to a neutron fluence value greater than 1×10^{21} n/cm² ($E > 1$ MeV) during the current ISI interval. Since the inspection frequency in the BWRVIP-76 report is based on fracture toughness values which are not consistent with the BWRVIP-100-A report, the NRC staff requests that the licensee address the following issue:

The inspection frequency and strategy that are specified in Section 3 of the BWRVIP-76 report require further evaluation taking into account the lower fracture toughness values that are specified in the BWRVIP-100-A report.

3. Dresden and Quad Cities (D/QC) Safety Evaluation Report, "NUREG-1796, Related to the License Renewal of the Dresden Nuclear Power Station, Units 2 and 3 and Quad Cities Nuclear Power Station, Units 1 and 2," license renewal application (LRA) commitment #9 in Appendix A of NUREG-1796 states that the licensee should implement the staff approved aging management program for the steam dryers at the D/QC units. In July 2009, the BWRVIP issued a staff approved topical report BWRVIP-139-A, "BWR Vessel and Internals Project, Steam Dryer Inspection and Flaw Evaluation Guidelines." The NRC staff requests that the licensee confirm that it will comply with the guidelines addressed in the BWRVIP-139-A report as per LRA commitment #9 in NUREG-1796.
4. Consistent with the LRA commitment #9, with respect to the aging management program related to the top guide, the licensee should confirm that it will comply with the inspection guidelines addressed in the BWRVIP-26-A and BWRVIP-183 reports.

A copy of this email will be placed into ADAMS and made publicly available.

Sincerely,

Blake Purnell, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
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U.S. Nuclear Regulatory Commission
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