



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 2, 2013

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION – CORRECTED RELIEF
REQUEST FROM THE REQUIREMENTS OF THE ASME CODE, RELIEF
REQUEST NO. I5R-05 FOR EXPANDED APPLICABILITY FOR USE OF ASME
CODE CASE N-661-1 (TAC NO. ME9492)

Dear Mr. Pacilio:

By letter dated April 18, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13092A401), the U.S. Nuclear Regulatory Commission (NRC) authorized Relief Request I5R-05 for Oyster Creek Nuclear Generating Station. Subsequently, the NRC staff was notified of typographical errors in the cover letter dated April 18, 2013, and the associated safety evaluation (SE). Specifically, the typographical errors identified included areas where I5R-01 was used instead of I5R-05, and an area where IST was used instead of ISI. The errors occurred on the first page of the cover letter, and the first page of the associated SE.

Corrected copies of page 1 of the cover letter and page 1 of the associated SE are enclosed, with revision bars indicating the areas of change. This correction does not change any of the conclusions in the SE associated with the Relief Request.

We apologize if this error has caused any inconvenience. If you have any questions regarding this matter, please contact the Senior Project Manager, John G. Lamb, at (301) 415-3100 or by e-mail at John.Lamb@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Meena Khanna", is located below the "Sincerely," text.

Meena Khanna, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures:
Corrected Cover Letter and SE page

cc w/enclosures: Distribution via Listserv

ENCLOSURE

Oyster Creek Nuclear Generating Station

Corrected Pages for Relief Request I5R-05 dated April 18, 2013



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April 18, 2013

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EXPANDED APPLICABILITY FOR USE OF ASME CODE CASE N-661-1
(TAC NO. ME9492)

Dear Mr. Pacilio:

By letter dated August 28, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12243A287), as supplemented by letter dated December 3, 2012 (ADAMS Accession No. ML12339A012), Exelon Generation Company (the licensee) submitted request for alternative (RFA) I5R-05 for the U.S. Nuclear Regulatory Commission's (NRC) approval. The licensee proposed an alternative to certain requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. RFA I5R-05 relates to the applicability of ASME Code Case N-661-1, "Alternative Requirements for Wall Thickness Restoration of Class 2 and 3 Carbon Steel Piping for Raw Water Service," which is limited up to the 2005 Addenda of the ASME Code. RFA I5R-05 is requested for the fifth 10-year inservice inspection (ISI) interval of the Oyster Creek Nuclear Generating Station (Oyster Creek) which commenced on January 15, 2013. The 2007 Edition through the 2008 Addenda of the ASME Code, Section XI, is the current Code of record at Oyster Creek. Specifically, pursuant to Title 10 of the *Code Federal Regulations* (10 CFR) Section 50.55a(a)(3)(i), the licensee requested to use the proposed alternative on the basis that the alternative provides an acceptable level of quality and safety.

The NRC staff has reviewed the subject request and has concluded, as set forth in the enclosed safety evaluation, that the proposed alternative described in RFA I5R-05 provides an acceptable level of quality and safety. Accordingly, the NRC staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(a)(3)(i), and is in compliance with the ASME Code's requirements.

Therefore, the NRC staff authorizes the alternative described in Relief Request I5R-05 for the fifth ISI interval at Oyster Creek, which began on January 15, 2013, and ends on January 14, 2023. All other ASME Code requirements for which relief was not specifically requested and approved in the subject request remain applicable, including a third party review by the Authorized Nuclear Inservice Inspector.



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

REQUEST FOR RELIEF I5R-05

FIFTH INSERVICE INSPECTION INTERVAL

OYSTER CREEK NUCLEAR GENERATING STATION

EXELON NUCLEAR

DOCKET NO. 50-219

1.0 INTRODUCTION

By letter dated August 28, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12243A287), as supplemented by letter dated December 3, 2012 (ADAMS Accession No. ML12339A012), Exelon Generation Company (the licensee) submitted request for alternative (RFA) I5R-05 for the U.S. Nuclear Regulatory Commission's (NRC) approval. The licensee proposed an alternative to certain requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. RFA I5R-05 relates to the applicability of ASME Code Case N-661-1, "Alternative Requirements for Wall Thickness Restoration of Class 2 and 3 Carbon Steel Piping for Raw Water Service," which is limited up to the 2005 Addenda of the ASME Code. RFA I5R-05 is requested for the fifth 10-year inservice inspection (ISI) interval of the Oyster Creek Nuclear Generating Station (Oyster Creek) which commenced on January 15, 2013. The 2007 Edition through the 2008 Addenda of the ASME Code, Section XI, is the current code of record at Oyster Creek.

Specifically, pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(a)(3)(i), the licensee proposed an alternative to repair or replacement (i.e., the applicability of ASME Code Case N-661-1 is extended to the 2007 Edition through 2008 Addenda for use in the Oyster Creek fifth 10-year ISI interval) for Class 2 and 3 carbon steel raw water piping systems, on the basis that the alternative provides an acceptable level of quality and safety.

2.0 REGULATORY EVALUATION

10 CFR 50.55a(g)(4) specifies that ASME Code Class 1, 2, and 3 components (including supports) must meet the requirements, except the design and access provisions and the preservice examination requirements, set forth in the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. The regulations require that inservice examination of components and system pressure tests conducted during the first 10-year interval and subsequent intervals comply with the

Enclosure

May 2, 2013

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Sincerely,

/ra/

Meena Khanna, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures:
Corrected Cover Letter and SE page

cc w/enclosures: Distribution via Listserv

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ADAMS Accession No.: ML13113A016

OFFICE	LPL1-2/PM	LPL1-2/LA *	EPNB/BC	LPL1-2/BC
NAME	JLamb	ABaxter	TLupold*	MKhanna
DATE	04/30/13	04/ 26 /13	03/27/13	05/02/13

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