

NRR-PMDAPEm Resource

From: DiFrancesco, Nicholas
Sent: Thursday, February 14, 2013 5:17 PM
To: Lisa.Simpson@exeloncorp.com
Cc: Wiebe, Joel; David.Gullott@exeloncorp.com; Purnell, Blake; DiFrancesco, Nicholas
Subject: February 14, 2013, Request for Additional Information RE: NRC Review of the LaSalle Unit 2 SLMCPR Change (TAC NO. ME9769)

Ms. Lisa Simpson,

By letters to the U.S. Nuclear Regulatory Commission (NRC) dated October 11, 2012, and January 17, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML12285A387 and ML130180301, respectively), Exelon Generation Company, LLC (the licensee) submitted a license amendment request proposing to modify the safety limit minimum critical power ratios contained in Section 2.1 of the technical specifications at LaSalle County Station (LSCS), Unit 2. The proposed changes support upcoming Cycle 15 operation of LSCS, Unit 2.

The NRC staff reviewed your submittals and has determined that additional information is required to complete the review. The specific information requests are provided below. A copy of this email is being placed in ADAMS as a publicly-available, official agency record.

Sincerely,

Nicholas DiFrancesco

Project Manager - LaSalle and Power Uprate Program
U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Division of Operating Reactor Licensing
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REQUEST FOR ADDITIONAL INFORMATION

EXELON GENERATION COMPANY, LLC

LASALLE COUNTY STATION, UNIT 2

LICENSE AMENDMENT REQUEST REGARDING

SAFETY LIMIT MINIMUM CRITICAL POWER RATIO

DOCKET NO. 50-374

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the January 17, 2013, Exelon Generation Company, LLC, (EGC) letter which provided responses to NRC staff RAI regarding changes to the LaSalle County Station (LSCS), Unit 2, Cycle 15, safety limit minimum critical power ratios (SLMCPRs). Following review of the RAI responses, the staff has the following questions for clarification:

1. Regarding the second question under RAI-02. Please provide technical justification for the magnitude of the increase of the SLMCPR values by 0.03 and 0.05 for two recirculation loop operation and single loop operation, respectively. What factors contribute to the change?
2. Please provide evaluation or design documents which demonstrate that the conclusion in 'GNF Response to RAI-03' are applicable to the LaSalle Unit 2, GNF2 fuel design with Areva Atrium-10 fuel for the following items:
 - Item 1) TGBLA fuel rod power calculation uncertainty for ATRIUM-10,
 - Item 2) the R-factor uncertainty for ATRIUM-10, and
 - Item 4) the bundle power uncertainty associated the core monitoring system applied to ATRIUM-10.
3. In follow-up to, "GNF Response to RAI-04-2," please provide analysis which demonstrate that items 1, 2, 3, and 4, for ATRIUM-10 nuclear fuel are applicable and will remain applicable to future cycles. Although, the licensee evaluated the expand application range change through a 50.59 evaluation based on a Columbia license amendment approval. Please provide technical justification and analysis which demonstrates the expanded application range is appropriate for the Atrium-10 fuel for LaSalle Unit 2. Please discuss whether the expanded range of application is being utilized for the proposed LaSalle Cycle 15 core design. Lastly, please confirm that the NRC Part 21 Log No. 2007-29-00 (ADAMS No. ML072830334) regarding ATRIUM-10 MCPR remains dispositioned for LaSalle Unit 2, Cycle 15.

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