

Proposed - For Interim Use and Comment



U.S. NUCLEAR REGULATORY COMMISSION DESIGN-SPECIFIC REVIEW STANDARD FOR mPOWER™ iPWR DESIGN

5.2.1.2 APPLICABLE CODE CASES

REVIEW RESPONSIBILITIES

Primary - Organization responsible for mechanical engineering reviews

Secondary - Organization responsible for component integrity reviews

I. AREAS OF REVIEW

Integral pressurized water reactors (iPWRs) generally incorporate the reactor core, reactor coolant pumps (if included in the design), and the pressurizer inside the reactor vessel. One or more steam generators may be inside the reactor vessel or directly connected to the reactor pressure vessel. This limits the size and length of piping connected to the reactor vessel and the number of mechanical components outside the reactor vessel. Connections to an iPWR reactor vessel typically include the steam system, the feedwater system, a charging and letdown system, safety relief valves, and an emergency core cooling system or other emergency safety feature systems.

The specific areas of review are as follows:

1. American Society of Mechanical Engineers (ASME) Code Cases contain alternatives to the requirements in the ASME *Boiler & Pressure Vessel Code* (B&PV Code) and ASME *Code for Operation and Maintenance of Nuclear Power Plants* (OM Code) for the design, fabrication, manufacturing, construction, installation, testing, examination, and inspection of nuclear power plant components within the scope of the ASME B&PV Code and ASME OM Code. The U.S. Nuclear Regulatory Commission (NRC) staff review of an iPWR application under 10 CFR Part 50 or Part 52 will determine the acceptability of ASME Code Cases specified in the applicant's technical submittal. The review will determine the acceptability of ASME Code Cases applied to ASME B&PV Code, Section III, Division 1, Subsection NB - Class 1 Components, Subsection NC - Class 2 Components, Subsection ND - Class 3 Components, and Subsection NE - Class MC Components. The review will also determine the acceptability of Code Cases applied to ASME B&PV Code, Section III, Division 1, Subsection NE - Class MC Components, Subsection NF - Component Supports, and Subsection NG - Core Support Structures, and ASME B&PV Code, Section III, Division 2, Concrete Containment. Further, the review will determine the acceptability of Code Cases that may be applied to ASME B&PV Code, Section XI, Division 1, Inservice Inspection, and the ASME OM Code, Inservice Testing.
2. Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC). For design certification (DC) and combined license (COL) applications, the staff reviews the applicant's proposed ITAAC associated with the structures, systems, and components (SSCs) related to this DSRS section in accordance with NUREG-0800 SRP Section 14.3, "Inspections, Tests, Analyses, and Acceptance Criteria." The staff recognizes that the

review of ITAAC cannot be completed until after the application has been reviewed against acceptance criteria contained in this DSRS section. Furthermore, the staff reviews the ITAAC to ensure that all SSCs in this area of review are identified and addressed as appropriate in accordance with SRP Section 14.3.

3. COL Action Items and Certification Requirements and Restrictions. For a DC application, the review will address COL action items and requirements and restrictions (e.g., interface requirements and site parameters).

For a COL application referencing a DC, a COL applicant must address COL action items (referred to as COL license information in certain DCs) included in the referenced DC. Additionally, a COL applicant must address requirements and restrictions (e.g., interface requirements and site parameters) included in the referenced DC.

4. The NRC regulations in 10 CFR 52.47(a)(27) and 52.79(a)(46) require DC and COL applicants, respectively, to submit a description of the plant-specific probabilistic risk assessment (PRA) and its results in their applications. Therefore, the NRC staff may incorporate risk insights into the review of DC and COL applications as described in this and other SRP and DSRS sections. When applying risk insights in its application review, the applicable technical branch will focus its application review on safety-related components categorized as having high risk significance. For safety-related components categorized as having low risk significance, the staff will rely on the applicant's certification of component qualification and programmatic requirements to provide reasonable assurance that the components satisfy the requirements in the applicable codes and standards as incorporated by reference in 10 CFR 50.55a, with confirmation of compliance with those regulatory requirements through ITAAC verification and inspection activities. In addition, the staff will evaluate the quality standards specified by the applicant for the design, fabrication, erection, inspection, and testing of components outside the scope of the codes and standards incorporated by reference in 10 CFR 50.55a, but determined to have high risk significance, commensurate with the importance of the safety function to be performed.

Review Interfaces

Other SRP and DSRS sections interface with this section as follows:

1. The organization responsible for component integrity will evaluate Code Cases pertaining to nondestructive testing under DSRS Sections 3.12 and 3.13.
2. The technical branch responsible for structural analysis reviews will evaluate Code Cases pertaining to ASME B&PV Code, Section III, Division 2, under SRP Sections 3.8.1, 3.8.3, and 3.8.5.
3. The organization responsible for component integrity will evaluate Code Cases pertaining to OM Code under DSRS Section 3.9.6.
4. Appropriate technical branches will evaluate other areas covered by ASME Code Cases as necessary.

II. ACCEPTANCE CRITERIA

Requirements

Acceptance criteria are based on meeting the relevant requirements of the following Commission regulations:

1. 10 CFR Part 50, Appendix A, General Design Criterion (GDC) 1, as it relates to the requirement that SSCs important to safety shall be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed;
2. 10 CFR 50.55a, as it relates to the rule that establishes minimum quality standards for the design, fabrication, erection, construction, testing, and inspection of certain components of IPWR nuclear power plants by requiring conformance with appropriate editions of specified industry codes and standards;
3. 10 CFR 52.47(b)(1), which requires that a DC application contain the proposed ITAAC that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a facility that incorporates the design certification has been constructed and will be operated in conformity with the design certification, the provisions of the Atomic Energy Act (AEA), and the U.S. Nuclear Regulatory Commission's (NRC's) regulations;
4. 10 CFR 52.79(a)(11), which requires a COL applicant to provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME B&PV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance; and
5. 10 CFR 52.80(a), which requires that a COL application contain the proposed inspections, tests, and analyses, including those applicable to emergency planning, that the licensee shall perform, and the acceptance criteria that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the combined license, the provisions of the Atomic Energy Act, and the NRC's regulations.

DSRS Acceptance Criteria

Specific DSRS acceptance criteria acceptable to meet the relevant requirements of the NRC's regulations identified above are as follows for review described in this DSRS section. The DSRS is not a substitute for the NRC's regulations, and compliance with it is not required. Identifying the differences between this DSRS section and the design features, analytical techniques, and procedural measures proposed for the facility, and discussing how the proposed alternative provides an acceptable method of complying with the regulations that underlie the DSRS acceptance criteria, is sufficient to meet the intent of 10 CFR 52.47(a)(9), "Contents of applications; technical information."

1. 10 CFR Part 50, Appendix A, GDC 1 requires that SSCs be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed. For those SSCs defined as safety-related, the NRC regulations specify special treatment requirements to provide reasonable assurance of

the capability of those SSCs to perform their safety-related functions. One special treatment requirement is that applicable components meet the requirements in the ASME B&PV Code and ASME OM Code as incorporated by reference in 10 CFR 50.55a. ASME Code Cases provide alternatives to specific requirements in the ASME B&PV Code and ASME OM Code. The NRC staff considers ASME Code Cases to satisfy the requirements in 10 CFR Part 50, Appendix A, GDC 1 to design, fabricate, erect, and test components to quality standards commensurate with the importance of the safety function to be performed, where those Code Cases have been accepted by the NRC staff. In addition to its application for safety-related components, the staff will apply this position in its review of nonsafety-related components that are categorized as having high risk significance, where risk insights are applied in the iPWR application review.

2. 10 CFR 50.55a requires that components of the Reactor Coolant Pressure Boundary be designed, fabricated, erected, and tested in accordance with the requirements for Class 1 components of Section III of the ASME B&PV Code. The Codes and Standards Rule also requires that pressure-retaining components of other fluid systems designated as Quality Group B or Quality Group C meet ASME B&PV Code requirements for Class 2 or Class 3 components, respectively. Components within the scope of ASME B&PV Code, Section XI, and ASME OM Code are subject to ISI and IST in accordance with ASME B&PV Code and OM Code, respectively, as incorporated by reference in 10 CFR 50.55a.

The NRC regulations in 10 CFR 50.55a incorporate by reference specific revisions of the following NRC regulatory guides (RGs) for the acceptability of ASME Code Cases:

- a. RG 1.84, "Design and Fabrication Code Case Acceptability, ASME Section III,." This guide lists those ASME B&PV Code, Section III, Code Cases oriented to design, fabrication, materials, and testing that are acceptable to the staff for implementation in the licensing of nuclear power plants.
- b. RG 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1." This guide lists those ASME B&PV Code, Section XI Code Cases that are acceptable to the staff for use in the inservice inspection of components and their supports, as described in the first numbered paragraph of subsection I, of this DSRS.
- c. RG 1.192, "Operation and Maintenance Code Case Acceptability, ASME OM Code." This guide lists ASME OM Code Cases oriented to operation and maintenance for nuclear power plant components that are acceptable to the staff for implementation in the licensing of nuclear power plants.

For Code Cases pertaining to ASME B&PV Code Section III, Division 2, as well as ASME Code Cases not covered in RG 1.84, 1.147, or 1.192, the iPWR applicant may submit a request for NRC authorization to use an alternative to the ASME Code requirements in accordance with 10 CFR 50.55a. The NRC staff will review the proposed alternative for authorization based on the following considerations as indicated in 10 CFR 50.55a:

- a. If the proposed Code Cases provide an acceptable level of quality and safety; or

- b. If compliance with the specified requirements of 10 CFR 50.55a would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.
- 3. 10 CFR 52.47(b)(1) requires that a DC application contain proposed ITAAC necessary and sufficient to assure the plant is built and will operate in accordance with the design certification. 10 CFR 52.80(a) requires that the COL identify the ITAAC necessary and sufficient to assure that the facility has been constructed and will be operated in conformity with the license. SRP Section 14.3 provides guidance for reviewing ITAAC. The requirements of 10 CFR 52.47(b)(1) and 10 CFR 52.80(a) will be met, in part, by identifying ITAAC of the top-level design features of applicable Code cases in the DC and COL applications, respectively.
- 4. 10 CFR 52.79(a)(11) requires that a COL applicant provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME B&PV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance. RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)," provides guidance for the content of COL applications for a description of the ISI and IST programs to meet 10 CFR 50.55a. The organizational staff responsible for component integrity reviews the descriptions of these programs and documents its review in the applicable SER sections.

Technical Rationale

The technical rationale for application of these acceptance criteria to the areas of review addressed by this DSRS section is discussed in the following paragraphs:

- 1. Compliance with GDC 1 requires that SSCs be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety functions to be performed.

DSRS Section 5.2.1.2 cites RGs 1.84, 1.147, and 1.192 for Code Cases applicable to ASME B&PV Code, Section III, Division 1, components and materials, and Section XI, Division 1, tests and inspections, and ASME OM Code for operation and maintenance of nuclear power plant components, that are acceptable to the staff.

Applicants may submit requests for authorization to use Code Cases for ASME B&PV Code, Section III, Division 2, and other ASME Code Cases not accepted in RGs 1.84, 1.147, and 1.192 as alternatives to ASME Code requirements in accordance with 10 CFR 50.55a. The NRC staff will prepare a safety evaluation describing its review of proposed alternatives to the ASME Code requirements.

The staff considers the application of Code Cases accepted in RGs 1.84, 1.147, and 1.192, or accepted in specific safety evaluations, to satisfy GDC 1 in providing adequate assurance that plant SSCs will perform acceptably, commensurate with the importance of their safety function.

- 2. Section 50.55a of 10 CFR requires that SSCs be designed, fabricated, erected, constructed, tested, and inspected to quality standards commensurate with the importance of the safety functions to be performed and allows the use of ASME Code Cases accepted in RGs 1.84, 1.147, and 1.192 incorporated by reference in 10 CFR 50.55a.

RGs 1.84, 1.147, and 1.192 as incorporated by reference in 10 CFR 50.55a identify Code Cases applicable to ASME B&PV Code, Section III, Division 1, components and materials, and Section XI, Division 1, tests and inspections, and ASME OM Code for operation and maintenance of nuclear plant components, that are acceptable to the NRC staff. As discussed above, the regulations in 10 CFR 50.55a allow applicants to submit requests for authorization to use Code Cases for ASME B&PV Code, Section III, Division 2, and other ASME Code Cases not accepted in RGs 1.84, 1.147, and 1.192 as alternatives to ASME Code requirements. The NRC staff will prepare a safety evaluation describing its review of proposed alternatives to the ASME Code requirements.

The staff considers the application of Code Cases accepted in RGs 1.84, 1.147, and 1.192, or accepted in specific safety evaluations, to satisfy 10 CFR 50.55a in providing adequate assurance that these SSCs will perform acceptably, commensurate with the importance of their safety function.

3. 10 CFR 52.47(b)(1) requires that a DC application contain proposed ITAAC necessary and sufficient to assure the plant is built and will operate in accordance with the design certification. 10 CFR 52.80(a) requires that the COL identify the ITAAC necessary and sufficient to assure that the facility has been constructed and will be operated in conformity with the license. SRP Section 14.3 provides guidance for reviewing ITAAC. The staff considers these requirements in 10 CFR 52.47(b)(1) and 10 CFR 52.80(a) to be sufficient to provide confidence in compliance with 10 CFR 50.55a in iPWR design certification and COL applications where accepted ASME Code Cases are applied.
4. 10 CFR 52.79(a)(11) requires that a COL applicant provide, in its safety analysis report, a description of the programs and their implementation necessary to ensure that the systems and components meet the requirements of the ASME B&PV Code and ASME OM Code in accordance with 10 CFR 50.55a at a level sufficient to enable the NRC to reach a final conclusion on all safety matters that must be resolved before COL issuance. RG 1.206 provides guidance for the content of COL applications for a description of the ISI and IST programs to meet 10 CFR 50.55a. The staff considers the requirements in 10 CFR 52.79(a)(11) and the guidance in RG 1.206 to be sufficient to provide confidence in compliance with 10 CFR 50.55a where accepted ASME Code Cases are applied.
5. Where risk insights are applied in the iPWR application review, the programmatic requirements listed in the acceptance criteria provide confidence that SSCs in the iPWR will perform their intended functions commensurate with their importance to safety. For example, Appendix A to 10 CFR Part 50 requires that SSCs be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed. 10 CFR Part 50, Appendix B, Quality Assurance Criteria, requires that a quality assurance program be established for SSCs classified as safety-related to provide reasonable assurance that those SSCs are available to perform their safety functions. The availability controls for the regulatory treatment for nonsafety systems (RTNSS) as discussed in RG 1.206 apply to nonsafety-related systems that are categorized as having high risk significance. The staff will apply these and other requirements to provide assurance that SSCs in the iPWR will perform their intended functions commensurate with their importance to safety. The staff considers the application of accepted ASME Code Cases can be used in satisfying the programmatic requirements in the acceptance criteria, where applicable.

III. REVIEW PROCEDURES

These review procedures are based on the identified DSRS acceptance criteria. For deviations from these acceptance criteria, the staff should review the applicant's evaluation of how the proposed alternatives provide an acceptable method of complying with the relevant NRC requirements identified in Subsection II.

1. Programmatic Requirements and Guidance - In accordance with the guidance in NUREG-0800 "Introduction," Part 2 as applied to this DSRS Section, the staff will review the programs proposed by the applicant to satisfy the following programmatic requirements. If any of the proposed programs satisfies the acceptance criteria described in Subsection II, it can be used to augment or replace some of the review procedures. It should be noted that the wording of "to augment or replace" applies to nonsafety-related risk-significant SSCs, but "to replace" applies to nonsafety-related nonrisk-significant SSCs according to the "graded approach" discussion in NUREG-0800 "Introduction," Part 2. Commission regulations and policy mandate programs applicable to SSCs. Examples of those programs and associated guidance follows:
 - 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants"
 - 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants"
 - 10 CFR 50.55a, "Codes and Standards"
 - Maintenance Rule, SRP Section 17.6 (SRP Section 13.4, Table 13.4, Item 17, RG 1.160, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants." and RG 1.182; "Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants".)
 - Quality Assurance Program, 10 CFR Part 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants", SRP Sections 17.3 and 17.5 (DSRS Section 13.4, Table 13.4, Item 16).
 - Technical Specifications (DSRS Section 16.0 and SRP Section 16.1) – including brackets value for DC and COL. Brackets are used to identify information or characteristics that are plant specific or are based on preliminary design information.
 - Reliability Assurance Program (SRP Section 17.4).
 - Regulatory Treatment of Non-safety Systems (SRP Section 19.3)
 - Initial Plant Test Program (RG 1.68, "Initial Test Programs for Water-Cooled Nuclear Power Plants," SRP Section 14.2, and DSRS Section 13.4, Table 13.4, Item 19).
 - ITAAC (SRP/DSRS Chapter 14).

2. In accordance with 10 CFR 52.47(a)(8),(21), and (22), and 10 CFR 52.79(a)(17) and (20), for new reactor license applications submitted under Part 52, the applicant is required to (1) address the proposed technical resolution of unresolved safety issues and medium- and high-priority generic safety issues which are identified in the version of NUREG-0933 current on the date up to 6 months before the docket date of the application and which are technically relevant to the design; (2) demonstrate how the operating experience insights have been incorporated into the plant design; and, (3) provide information necessary to demonstrate compliance with any technically relevant portions of the Three Mile Island requirements set forth in 10 CFR 50.34(f), except paragraphs (f)(1)(xii), (f)(2)(ix), and (f)(3)(v). These cross-cutting review areas should be addressed by the reviewer for each technical subsection and relevant conclusions documented in the corresponding safety evaluation report (SER) section.
3. For reviews of DC and COL applications under 10 CFR Part 52, the NRC staff verifies that Section 5.2.1.2 of the applicant's submitted documentation specifies the use of ASME Code Cases that are included in revisions to RGs 1.84, 1.147, and 1.192 incorporated by reference in 10 CFR 50.55a. The staff also checks the table provided by the iPWR applicant identifying the applied ASME Code Cases for compliance with RGs 1.84, 1.147, and 1.192.
4. For ASME Code Cases pertaining to ASME B&PV Code, Section III, Division 2, as well as for other Code Cases not accepted in RG 1.84, 1.147, or 1.192, the NRC staff reviews the request by the iPWR applicant to apply those Code Cases as an alternative to the ASME Code requirements in accordance with 10 CFR 50.55a. The NRC staff will review ASME Code Cases proposed by the iPWR applicant as alternatives to the ASME Code requirements for authorization based on the following considerations as specified in 10 CFR 50.55a:
 - a. Does the proposed Code Case provide an acceptable level of quality and safety?
or
 - b. Does compliance with the specified requirements of 10 CFR 50.55a result in hardship or unusual difficulty without a compensating increase in the level of quality and safety?

For review of a DC application, the reviewer should follow the above procedures to verify that the design, including requirements and restrictions (e.g., interface requirements and site parameters), set forth in the DC applicant's design control document (DCD) or Final Safety Analysis Report (FSAR) meets the acceptance criteria. The reviewer should also consider the appropriateness of identified COL action items. The reviewer may identify additional COL action items that should be added to the DC applicant's DCD or FSAR.

For review of a COL application, the scope of the review is dependent on whether the COL applicant references a DC, an early site permit or other NRC approvals (e.g., manufacturing license, site suitability report or topical report).

For review of both DC and COL applications, SRP Section 14.3 should be followed for the review of ITAAC. The review of ITAAC cannot be completed until after the completion of this section.

IV. EVALUATION FINDINGS

The reviewer verifies that the applicant has provided sufficient information and that the staff's technical review and analysis support conclusions of the following type to be included in the staff's safety evaluation report. The reviewer also states the bases for those conclusions.

The NRC staff finds that the ASME Code Cases specified in the iPWR application to be applied in the design, fabrication, manufacturing, construction, installation, testing, examination, and inspection of components within the scope of the ASME B&PV Code and ASME OM Code are acceptable for use in accordance with the requirements of 10 CFR 50.55a and 10 CFR Part 50, Appendix A, GDC 1; and the guidance provided in RGs 1.84, 1.147, and 1.192. For ASME Code Cases not accepted in RGs 1.84, 1.147, and 1.192, the NRC staff has found that the applicant's request to apply those Code Cases specified in its technical submittal as alternatives to the Code requirements is authorized in accordance with 10 CFR 50.55a. The staff concludes that compliance with these Code Cases will result in a component quality level commensurate with the importance of their safety function and constitutes an acceptable basis for satisfying the requirements of 10 CFR 50.55a and 10 CFR Part 50, Appendix A, GDC 1.

For DC and COL reviews, the findings will also summarize the staff's evaluation of requirements and restrictions (e.g., interface requirements and site parameters) and COL action items relevant to this DSRS section.

In addition, to the extent that the review is not discussed in other SER sections, the findings will summarize the staff's evaluation of the ITAAC, including design acceptance criteria, as applicable.

V. IMPLEMENTATION

The staff will use this DSRS section in performing safety evaluations of mPower™-specific DC, or COL, applications submitted by applicants pursuant to 10 CFR Part 52. The staff will use the method described herein to evaluate conformance with Commission regulations.

Because of the numerous design differences between the mPower™ and large light-water nuclear reactor power plants, and in accordance with the direction given by the Commission in SRM- COMGBJ-10-0004/COMGEA-10-0001, "Use of Risk Insights to Enhance the Safety Focus of Small Modular Reactor Reviews," dated August 31, 2010 (ML102510405), to develop risk-informed licensing review plans for each of the small modular reactor reviews including the associated pre-application activities, the staff has developed the content of this DSRS section as an alternative method for mPower™-specific DC, or COL submitted pursuant to 10 CFR Part 52 to comply with 10 CFR 52.47(a)(9), "Contents of applications; technical information."

This regulation states, in part, that the application must contain "an evaluation of the standard plant design against the Standard Review Plan (SRP) revision in effect 6 months before the docket date of the application." The content of this DSRS section has been accepted as an alternative method for complying with 10 CFR 52.47(a)(9) as long as the mPower™ DCD FSAR does not deviate significantly from the design assumptions made by the NRC staff while preparing this DSRS section. The application must identify and describe all differences between the standard plant design and this DSRS section, and discuss how the proposed alternative provides an acceptable method of complying with the regulations that underlie the DSRS acceptance criteria. If the design assumptions in the DC application deviate significantly from the DSRS, the staff will use the SRP as specified in 10 CFR 52.47(a)(9). Alternatively, the staff

may supplement the DSRS section by adding appropriate criteria in order to address new design assumptions. The same approach may be used to meet the requirements of 10 CFR 52.79(a)(41) for COL applications.

VI. REFERENCES

1. 10 CFR Part 50, Appendix A, General Design Criterion 1, "Quality Standards and Records."
2. 10 CFR 50.55a, "Codes and Standards."
3. *ASME Boiler and Pressure Vessel Code*, "Nuclear Power Plant Components," American Society of Mechanical Engineers.
4. *ASME Boiler and Pressure Vessel Code*, "Code Cases: Nuclear Components," American Society of Mechanical Engineers.
5. *ASME Code for Operation and Maintenance of Nuclear Power Plants*, American Society of Mechanical Engineers.
6. RG 1.68, "Initial Test Programs for Water-Cooled Nuclear Power Plants."
7. RG 1.84, "Design and Fabrication Code Case Acceptability, ASME Section III, Division 1."
8. RG 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1."
9. RG 1.160, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants."
10. RG 1.182, "Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants."
11. RG 1.192, "Operation and Maintenance Code Case Acceptability, ASME OM Code."
12. RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)."
13. RG 1.215, "Guidance for ITAAC Closure Under 10 CFR Part 52."