



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

August 9, 2012

Mr. Mano Nazar
Executive Vice President
and Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

**SUBJECT: NOTIFICATION OF SAINT LUCIE NUCLEAR PLANT - NRC COMPONENT
DESIGN BASES INSPECTION - INSPECTION REPORT
05000335/2012008 AND 05000389/2012008**

Dear Mr. Nazar:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Saint Lucie Nuclear Plant during the weeks of October 22 – October 26, November 5 - November 9, and November 26 – November 30, 2012. Robert Patterson, a Reactor Inspector from the NRC's Region II Office, will lead the inspection team. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued December 6, 2010.

The inspection will evaluate the capability of risk significant/low margin components to function as designed and to support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on August 2, 2012, Mr. Patterson confirmed with Kathy Rydman of your staff, arrangements for an information- gathering site visit, and the three-week onsite inspection. The schedule is as follows:

- Information-gathering visit: Week of October 1 - 5, 2012
- Onsite weeks: October 22 – October 26, November 5 - November 9, and November 26 – November 30, 2012

The purpose of the information-gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. John Hanna, a Region II Senior Reactor Analyst, will accompany Mr. Patterson during the information-gathering visit to review probabilistic risk assessment data and identify risk significant components that will be examined during the inspection.

The Enclosure lists documents that will be needed prior to the information-gathering visit. Please provide the referenced information to the Region II office by September 25, 2012. Contact Mr. Patterson with any questions concerning the requested information. The inspectors

will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

Additional documents will be requested during the information-gathering visit. The additional information will need to be available to the team in the Region II office prior to the inspection team's preparation week of October 15, 2012. Mr. Patterson will also discuss the following inspection support administrative details: availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection; method of tracking inspector requests during the inspection; licensee computer access; working space; arrangements for site access; and other applicable information.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its Enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Patterson at 404-997-4667 or me at 404-997-4530.

Sincerely,

/RA by Shane Sandal Acting For/

Rebecca Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No: 50-335 and 50-389
License No: DPR-67 and NPF-16

cc w/encl: (See next page)

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Sincerely,

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Rebecca Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No: 50-335, 50-389
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cc w/encl: (See next page)

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X SUNSI REVIEW COMPLETE X FORM 665 ATTACHED

OFFICE	RII:DRS	RII:DRS					
SIGNATURE	RNP1	/RA SRS5 for RLN1/					
NAME	R. Patterson	R. Nease					
DATE	8/ 6 /2012	8/ 9 /2012					
E-MAIL COPY	YES NO	YES NO					

OFFICIAL RECORD COPY DOCUMENT NAME: S:\DRS\ENG BRANCH 1\LETTERS TO LICENSEES\ST LUCIE CDBI
NOTIFICATION LETTER (RP).DOCX

cc w/encl:

Joseph Jensen
Site Vice President
St. Lucie Nuclear Plant
Electronic Mail Distribution

Paul Freeman
Vice President
Organizational Effectiveness
Florida Power & Light Company
Electronic Mail Distribution

Peter Wells
Vice President
Outage Support CFAM
Florida Power & Light Company
Electronic Mail Distribution

Robert J. Hughes
Plant General Manager
St. Lucie Nuclear Plant
Electronic Mail Distribution

Daniel D. DeBoer
Operations Site Director
St. Lucie Nuclear Plant
Electronic Mail Distribution

Eric Katzman
Licensing Manager
St. Lucie Nuclear Plant
Electronic Mail Distribution

Larry Nicholson
Director
Licensing
Florida Power & Light Company
Electronic Mail Distribution

Alison Brown
Nuclear Licensing
Florida Power & Light Company
Electronic Mail Distribution

Michael Baughman
Training Manager
St. Lucie Nuclear Plant
Electronic Distribution

Mitch S. Ross
Vice President and General
Counsel Nuclear
Florida Power & Light Company
Electronic Mail Distribution

Cynthia Becker
(Acting) Chief
Florida Bureau of Radiation Control
Department of Health
Electronic Mail Distribution

Bryan Koon
Director
Florida Division of Emergency Management
Electronic Mail Distribution

Donna Calabrese
Emergency Preparedness Manager
St. Lucie Plant
Electronic Mail Distribution

J. Kammel
Radiological Emergency Planning
Administrator
Department of Public Safety
Electronic Mail Distribution

Senior Resident Inspector
St. Lucie Nuclear Plant
U.S. Nuclear Regulatory Commission
P.O. Box 6090
Jensen Beach, FL 34957-2010

Faye Outlaw
County Administrator
St. Lucie County
Electronic Mail Distribution

Jack Southard
Director
Public Safety Department
St. Lucie County
Electronic Mail Distribution

INFORMATION REQUEST FOR SAINT LUCIE NUCLEAR PLANT COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM (or FTP site, Sharepoint, etc.) The CDROM (or website) should be indexed and hyperlinked to facilitate ease of use.

1. From your most-recent Probabilistic Safety Analysis (PSA) excluding external events and fires:
 - a. Two risk rankings of components from your site-specific PSA
 - i) one sorted by Risk Achievement Worth (RAW)
 - ii) the other sorted by Birnbaum Importance
 - b. A list of the top 500 cutsets
2. From your most-recent Probabilistic Safety Analysis (PSA) including external events and fires:
 - a. Two risk rankings of components from your site-specific PSA
 - i) one sorted by Risk Achievement Worth (RAW)
 - ii) the other sorted by Birnbaum Importance
 - b. A list of the top 500 cutsets
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide human reliability worksheets for these items.
4. List of time critical operator actions with a brief description of each action.
5. List of Emergency and Abnormal Operating Procedures revised (significant) since May 26, 2009, with a brief description of each revision.
6. List of components with low design margins (e.g., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, motor operated valve risk-margin rankings, etc.) and associated evaluations, or calculations.
7. List of station operating experience evaluations/reviews performed and documented in the station's corrective action program for industry events and safety related equipment failures/vulnerabilities [as communicated by NRC generic communications, industry communications, 10 CFR part 21 Notifications, etc.] since May 26, 2009.
8. List and brief description of safety related SSC design modifications implemented since May 26, 2009.
9. List and brief description of common-cause component failures that have occurred since May 26, 2009.

Enclosure

10. List and brief description of operability evaluations completed since May 26, 2009.
11. List of equipment on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address the issue(s).
12. List and brief description of equipment currently in degraded or nonconforming status as described in RIS 05-020.
13. List and reason for equipment classified in maintenance rule (a)(1) status since May 26, 2009, to present.
14. Copies of System Descriptions (or the like design basis documents) for Safety-Related Systems.
15. Copy of UFSAR(s).
16. Copy of Technical Specification(s).
17. Copy of Technical Specifications Bases.
18. Copy of Technical Requirements Manual(s).
19. List and brief description of Root Cause Evaluations that have been performed since May 26, 2009.
20. In-service Testing Program Procedure(s).
21. Corrective Action Program Procedure(s).
22. One line diagram of electrical plant (electronic and full size – hard copy week of October 1).
23. Index and legend for electrical plant one-line diagrams.
24. Primary AC calculation(s) for safety-related buses.
25. Primary DC calculation(s) for safety-related buses.
26. PI&D's for ECCS systems (electronic and 1/2 size – hard copy week of October 1).
27. Index and Legend for PI&Ds.
28. Copy of design bases documents for ECCS systems.
29. Copy of Operability determination procedure(s).
30. Copies of condition reports associated with findings from previous CDBI (if applicable).

31. Index (procedure number, titles, and current revision) of station Emergency Operating Procedures (EOPs), Abnormal Operating Procedures (AOPs), and Annunciator Response Procedures (ARPs).
32. Contact information for a person to discuss PRA information prior to the information-gathering trip (name, title, phone number, and e-mail address).

LIST OF ACRONYMS

AC	Alternating Current
AOP	Abnormal Operating Procedure
ARP	Annunciator Response Procedure
CDBI	Component Design Basis Inspection
DC	Direct Current
ECCS	Emergency Core Cooling System
EOP	Emergency Operating Procedure
FTP	File Transfer Protocol
NRC	Nuclear Regulatory Commission
OE	Operating Experience
P&ID	Piping and Instrumentation Diagram
PRA	Probabilistic Risk Assessment
PSA	Probabilistic Safety Assessment
RAW	Risk Achievement Worth
RIS	Regulatory Issue Summary
SSC	Structure, System or Component
UFSAR	Updated Final Safety Analysis Report