

July 26, 2012

MEMORANDUM TO: Richard Correia, Director
Division of Risk Analysis
Office of Nuclear Regulatory Research

THRU: Kevin Coyne, Chief */RA/*
Probabilistic Risk Assessment Branch
Division of Risk Analysis
Office of Nuclear Regulatory Research

FROM: Donald M. Helton */RA/*
Probabilistic Risk Assessment Branch
Division of Risk Analysis
Office of Nuclear Regulatory Research

SUBJECT: NOTICE OF SECOND CLOSED MEETING TO DISCUSS
ONGOING OFFICE OF NUCLEAR REGULATORY RESEARCH
CONFIRMATORY LEVEL 1 PROBABILISTIC RISK
ASSESSMENT SUCCESS CRITERIA ANALYSES

DATE AND TIME: Friday, August 10th, 2012
11:00 a.m.–12:00 p.m. Eastern Time

LOCATION: Telephone Bridge

PURPOSE: For NRC Office of Nuclear Regulatory Research staff and Exelon staff to discuss ongoing confirmatory Level 1 probabilistic risk assessment (PRA) success criteria analyses for the Byron Generating Station nuclear power plant. Meeting topics will include discussion of (1) boundary conditions and qualitative results for sequence timing and success criteria aspects of selected loss of direct current (DC) bus 111 sequences and (2) boundary conditions for upcoming calculations related to steam generator tube rupture. These discussions and analyses do not relate to any ongoing or anticipated regulatory actions, but rather, are to confirm specific underlying modeling aspects in the agency's Standardized Plant Analysis Risk models (a continuation of an activity described further in NUREG-1953, "Confirmatory Thermal-Hydraulic Analysis to Support Specific Success Criteria in the Standardized Plant Analysis Risk Models—Surry and Peach Bottom"). Additional details are provided in the meeting agenda.

PARTICIPANTS: NRC Offices
Office of Nuclear Regulatory Research
Office of Nuclear Reactor Regulation
Region 3

Outside Organizations
Exelon Corporation
Erin Engineering

CATEGORY:

This is a **noticed, but closed meeting**: The meeting has been closed for the following reasons: (a) there has been no public interest expressed in past public activities related to this work, (b) the meeting is an information exchange not related to any specific regulatory decision, and (c) closure of the meeting facilitates the discussion of facility details that are not in the public domain (e.g., emergency operating procedures). Should a member of the public wish to participate in any future discussions between NRC and Exelon associated with this project, please contact the NRC staff meeting contact listed below.

CONTACT:

Donald Helton, RES/DRA/PRAB
301-251-7594
Donald.Helton@nrc.gov

ADMINISTRATIVE:

A draft agenda for the meeting is attached to this notice.

The NRC will provide telephone bridge line information prior to the meeting date.

Enclosure:
Draft Meeting Agenda

Outside Organizations
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OFFICE	RES/DRA/PRAB	Tech Editor	RES/DRA/PRAB	RES/DRA
NAME	D. Helton	C. Hsu (via email)	K. Coyne	R. Correia
DATE	7/20/12	7/23/12	7/20/12	7/26/12

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DRAFT MEETING AGENDA

DISCUSS ONGOING U.S. NUCLEAR REGULATORY COMMISSION OFFICE OF NUCLEAR REGULATORY RESEARCH CONFIRMATORY LEVEL 1 PROBABILISTIC RISK ASSESSMENT SUCCESS CRITERIA ANALYSES

TELEPHONE BRIDGE: TO BE ANNOUNCED

11:00 am ET/10:00 CT	Role Call	All
11:05 am ET/10:05 CT	Brief Refresher on Project Impetus and History	Helton
11:10 am ET/10:10 CT	Overview of Loss of Direct Current Bus-111	Helton
	Qualitative Results	

The modeled sequence involves complete loss of direct current bus 111 and independent loss of diesel-driven auxiliary feedwater, leading to bleed and feed operation. The varied parameters for this scenario are the following: (1) the available high-pressure injection system (one charging train or one safety injection train), (2) the time at which operators begin bleed and feed operation (20 minutes versus 40 minutes versus never), and (3) whether or not the pressurizer pilot-operated relief valve sticks open due to excessive cycling.

11:30 am ET/10:30 CT	Discussion of Steam Generator Tube Rupture Boundary Conditions	Corson
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The modeled sequence involves a steam generator tube rupture at full power in which the following parameters are varied: (1) tube leakage area (0.5 to 2.0 double-ended tube rupture equivalents), (2) emergency core cooling system (ECCS) availability (minimal versus maximum), and (3) behavior of the atmospheric relief valves due to cycling. Auxiliary feedwater is available. Some underlying assumptions to be discussed are reactor trip on pressurizer level (procedure BOA-SEC-8), timing of main steam isolation valve closure, and securing of ECCS (procedure BEP-3).

11:50 am ET/10:50 CT	Questions/General Discussion	All
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The phone bridge will be open until 12:30 p.m. ET / 11:30 a.m. CT, in the case that the discussion runs long.