



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 19, 2012

Vice President, Operations  
Entergy Nuclear Operations, Inc.  
Indian Point Energy Center  
450 Broadway, GSB  
P.O. Box 249  
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 - REQUEST FOR  
ADDITIONAL INFORMATION REGARDING RELIEF REQUEST IP3-ISI-RR-05  
(TAC NO. ME7940)

Dear Sir or Madam:

By letter dated January 30, 2012 (Agencywide Documents Access and Management System Accession No. ML12039A253), Entergy Nuclear Operations, Inc., the licensee, submitted Relief Request IP3-ISI-RR-05 for the Indian Point Unit 3 fourth 10-year inservice inspection (ISI) interval. The relief request is for the application of Code Case N-716, "Alternative Piping Classification and Examination Requirements," to implement a risk informed/safety based ISI as an alternative to the ASME Section XI ISI requirements.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). Based on our discussions we understand that a response to the RAI will be provided within 45 days of the date of this letter.

Please contact me at (301) 415-1364 if you have any questions on this issue.

Sincerely,

A handwritten signature in black ink, reading "Douglas V. Pickett".

Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure:  
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION  
REGARDING USE OF RISK-INFORMED INSERVICE INSPECTION PROGRAM  
ENTERGY NUCLEAR OPERATIONS, INC.  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3  
DOCKET NO. 50-286

By letter dated January 30, 2012, Entergy Nuclear Operations, Inc., the licensee, submitted Relief Request IP3-1SI-RR-05, "Proposed Alternative to Use ASME Code Case N-716," (Agencywide Document Access and Management System Accession Number ML12039A253) for Nuclear Regulatory Commission review and authorization. In order to complete our review, the staff requests the following information.

1. The submittal indicates that a full-scope peer review was performed in 2010 for the internal events probabilistic risk assessment (PRA) and gap or self assessments have been performed periodically since this peer review. The application also indicates that the PRA model has been revised periodically and some of these revisions included changes to address findings from the past peer review and self assessments.
  - a. If any changes, since the independent full-scope peer review, are characterized as a PRA upgrade per ASME/ANS-RA-Sa-2009, please identify if a focused-scope peer review was performed for these changes consistent with the guidance in American Society of Mechanical Engineers/American Nuclear Society [ASME/ANS]-RA-Sa-2009, as endorsed by Regulatory Guide 1.200, and describe any findings from that focused-scope peer review and the resolution of these findings for this application.
  - b. If a focused-scope peer review has not been performed for changes characterized as a PRA upgrade, please describe what actions will be implemented to address this review deficiency and when the application will be supplemented to describe any findings from that focused-scope peer review and the resolution of these findings for this application.
2. For finding 6-11, the licensee indicated that the finding was resolved because "a list of internal flooding sources has been developed and will be included in a new Table 4.2.1.1 in the final update report." Please confirm that the final update report has been issued with the list of internal flooding sources developed and included.
3. For finding 6-1, the peer review team suggests performing rigorous evaluation/justification of the condensate storage tank (CST) inventory to support 24-hour Auxiliary Feed water System (AFW) operation because of discrepancy between information provided by the licensee in documentation and a Modular Accident Analysis Program (MAAP) analysis. The submittal asserts that the documentation has been updated to remove discrepancies. Considering the negative impact that an inadequate

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supply of CST inventory could have on the mitigating capabilities of AFW following a transient involving a loss of inventory or reduction in mitigation capabilities of core cooling, please confirm the model documentation has been modified indicating the CST inventory and its suitability to support a 24 hour mission time to support AFW operation.

4. The request for alternative states that Alloy 82/182 welds will be examined under the requirements of Code Case N-770-1, as required and conditioned by Title 10 of the *Code of Federal Regulations*, Part 50, Paragraph 55a(g)(6)(ii)(F), once the program has been formally implemented in 2013. Code Case N-770-1 has already been implemented and baseline examinations are due by the first refueling outage beginning after January 20, 2012. Please indicate if you are in compliance with the requirements of Code Case N-770-1.

July 19, 2012

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Sincerely,

/ra/

Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
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**ADAMS ACCESSION NO.: ML12200A056**

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