

1.6 Topical Reports and Other Documents

Table 1.6-1 is a list of all topical reports and any other reports or documents which are referenced in Tier 2 and which contain information utilized for the ABWR.

Table 1.6-1 Referenced Reports

Report No.	Title	Tier 2 Section No.
22A7007	"GESSAR II, 238 Nuclear Island, BWR/6 Standard Plant, General Electric Company", March 1980, & Amendments 1-21.	19.2 19E.2 19E.3
4401-4	"Nuclear Energy QA Program Description", Toshiba Corporation, Rev. 4	17.1
A98-1101-0007	"Generic Criteria for High Frequency Cutoff of BWR Equipment", June 2009.	3.9
7A25-3803-0006	"Geotechnical Site Parameters for Advanced Boiling Water Reactor (ABWR) Standard Plant Design", Revision 0, April 2009.	3A
7A31-0903-0001	"The Evaluation Report for Net Positive Suction Head of Pump in Emergency Core Cooling System", Toshiba Corporation, Revision 1, February 2010.	6C
7A31-0903-0002	"The Supplementary Document for the Head Loss Evaluation Report of Japanese ABWR ECCS Suction Strainer", Toshiba Corporation, Revision 3, February 2010.	6C
7A31-0903-0003	"The Evaluation Example of the Head Loss of the ECCS Suction Stainer and Pipe in the ECCS Pump Run-out Flow Condition", Toshiba Corporation, Revision 1, June 2009.	6C
APED-5750	"Design and Performance of General Electric Boiling Water Reactor Main Steam Line Isolation Valves", General Electric Company, Atomic Power Equipment Department, March 1969.	5.4
CDCC-2009-100118	"Occupational Radiation Exposure Experience for ABWRs in Japan".	12.4
CDCC-2009-100119	"Evaluation of Expected Annual Radiation Exposure at an ABWR".	12.4
CENPD-285-P-A	"Fuel Rod Design Methods for Boiling Water Reactors", July 1996.	4.2
CENPD-287-P-A	"Fuel Assembly Mechanical Design Methodology for Boiling Water Reactors", July 1996.	4.2
CENPD-288-P-A	"ABB Seismic/LOCA Evaluation Methodology for Boiling Water Reactor Fuel," July 1996."	3.9
CENPD-292-P-A	"BISON - One Dimensional Dynamic Analysis Code for Boiling Water Reactors: Supplement 1 to Code Description and Qualification", July 1996.	15.0
CENPD-300-P-A	"Reference Safety Report for Boiling Water Reactor Reload Fuel", July 1996.	4.1 4.3 4.4 16.2 16.3

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
CENPD-390-P-A	"The Advanced PHOENIX and POLCA Codes for Nuclear Design of Boiling Water Reactors", December 2000.	4.3 4A 15.0 16.3
NEDC-30259	H.A. Careway, D.B. Townsend, B.W. Shaffer, "A Technique for Evaluation of BWR MSIV Leakage Contribution to Radiological Dose Rate Calculations", September 1983.	20.3
NEDC-31393	"ABWR Containment Horizontal Vent Confirmatory Test, Part I", March 1987.	3B
NEDC-31858P	Louis Lee, "BWROG Report for Increasing MSIV Leakage Rate Limits and Elimination of Leakage Control System", 1991	15.6
NEDC-32267P	"ABWR Project Application Engineering Organization and Procedures Manual", December 1993.	17.1
NEDE-20566P	"General Electric Company, Analytical Model for Loss-of-Coolant Analysis in Accordance with 10CFR50, Appendix K", November 1975.	3.9
NEDE-21471-1	L. Lasher, et. al, "Analytical Model for Estimating Drag Forces on Rigid Submerged Structures Caused Supplement for X-Quencher Air Discharge", October 1979.	3B
NEDE-21544-P	R.J Ernst, et. al., "Mark II Pressure Suppression Containment Systems: An Analytical Model of the Pool Swell Phenomenon", December 1977.	19B
NEDE-22277-P-I	G. A. Watford, "Compliance of the GE BWR Fuel Design to Stability Licensing Criteria", October 1984.	20.3
NEDE-24057	"Assessment of Reactor Internals Vibration in BWR/4 and BWR/5 Plants", November 1977.	3.9
NEDE-24302-P	"Mark II Containment Program, Generic Chugging Load Definition Report", April 1981.	3B
NEDE-24326-1-P	"General Electric Environmental Qualification Program", Proprietary Document, January 1983.	3.9 3.11

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
NEDE-24988-P	"Analysis of Generic BWR Safety/Relief Valve Operability Test Results", October 1981.	1A
NEDE-25100-P	"Mark II Containment Supporting Program, Caorso SRV Discharge Tests Phase I Test Report", May 1979.	3B
NEDE-25118	"Mark II Containment Supporting Program, Caorso SRV Discharge Tests Phase II ATR", August 1979.	3B
NEDE-25273	F. T. Dodge, "Scaling Study of the General Electric Pressure Suppression Test Facility - Mark III Long Range Program, Task 2.2.1", SwRI, March 1980. (Proprietary)	3B
NEDE-31096-A	"GE Licensing Topical Report ATWS Response to NRC ATWS Rule 10CFR 50.62", February 1987.	19B.2 7.4
NEDM-20609-01	P.P. Stancavage and D.G. Abbott, "Liquid Discharge Doses LIDSR Code", August 1976.	12.2
NEDO-10029	"An Analytic Study on Brittle Fracture of GE-BWR Vessel Subject to the Design Basis Accident", June 1969.	5.3 20.3
NEDO-10299A	H.T. Kim, "Core Flow Distribution in a Modern BWR as Measured in Monticello", October 1976.	4.4
NEDO-10585	F.G. Brutchscy, et al., "Behavior of Iodine in Reactor Water During Plant Shutdown and Startup", August 1972.	15.2
NEDO-10722A	H.T. Kim, "Core Flow Distribution in a Large Boiling Water Reactor as Measured in Quad Cities Unit 1", August 1976.	4.4
NEDO-10871	J.M. Skarpelos and R.S. Gilbert, "Technical Derivation of BWR 1971 Design Basis Radioactive Material Source Terms", March 1973.	11.1
NEDO-11209-04-A	"GE Nuclear Energy Quality Assurance Program Description", the latest NRC-accepted version.	17.0 17.1 19A 20.3
NEDO-20206	D.R. Rogers, "BWR Turbine Equipment N-16 Radiation Shielding Studies", December 1973.	12.2

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
NEDO-20533	W.J.Bilanin, "The GE Mark III Pressure Suppression Containment Analytical Model", June 1974.	6.2 20.3
NEDO-20533-1	W.J.Bilanin, "The GE Mark III Pressure Suppression Containment Analytical Model", Supplement 1, September 1975.	6.2
NEDO-20804	"Atmospheric Dispersion CHIQUO2 Function", February 1979.	20.3
NEDO-21052	F.J. Moody, "Maximum Discharge Rate of Liquid-Vapor Mixtures from Vessels", General Electric Company, September 1975.	6.2
NEDO-21143-1	H. Careway, V. Nguyen, and P. Stancavege, "Radiological Accident-The CONACO3 CODE", December 1981.	15.2 15.6 15.7 20.3
NEDO-21159	"Airborne Releases from BWRs for Environmental Impact Evaluations" - Amendment 2 - Iodine.	12.2
NEDO-21159	"Airborne Releases from BWRs for Environmental Impact Evaluations", March 1976.	11.1 12.2
NEDO-21215	"Brunswick Steam Electric Plant Unit 1 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations", March 1976.	4.4
NEDO-21471	F. Moody, "Analytical Model for Estimating Drag Forces on Rigid Submerged Structures Caused by a LOCA", September 1977.	3B
NEDO-21778-A	"Transient Pressure Rises Affecting Fracture Toughness Requirements for Boiling Water Reactors", December 1978.	20.3
NEDO-21231	"Banked Position Withdrawal Sequence", January 1977.	16.3
NEDO-24057	"Assessment of Reactor Internals Vibration in BWR/4 and BWR/5 Plants", November 1977.	3.9
NEDO-24708	P. W. Marriot, "Additional Information Required for NRC Staff Generic Report on Boiling Water Reactors", August 1979.	1A 7.3
NEDO-25132A	E. W. Bradley, "Gamma & Beta Dose to Man from Noble Gas Release to the Atmosphere GEMAN Code", April 1980.	12.2

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
NEDO-25153	L. E. Lasher, "Analytical Model for Estimating Drag Forces on Rigid Structures Caused by Steam Condensation and Chugging", July 1979.	3B
NEDO-25257	E. W. Bradley and V. D. Nguyen, "Radiation Exposure from Airborne Effluents-the REFAE Code", July 1980.	12.2
NEDO-30832	J.E. Torbeck, "Elimination of Limit on BWR Suppression Pool Temperature for SRV Discharge With Quenchers", December 1984.	3B 20.3
NEDO-31331	Gerry Burnette, "BWR Owner's Group Emergency Procedure Guidelines", March 1987.	1A 18A 18B 20.3
NEDO-31439	C. VonDamm, "The Nuclear Measurement Analysis & Control Wide Range Neutron Monitoring System (NUMAC-WRNMS)", May 1987.	20.3
NEDO-31960-A	Glen Watford, "BWR Owners' Group Long-Term Stability Solutions Licensing Methodology", June 1991.	15.9
NEDO-31960-A Supplement 1	"BWR Owners' Group Long-Term Stability Solutions Licensing Methodology Supplement 1", March 1992.	15.9
NEDO-32465-A	"BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology for Reload Applications", August 1996.	15.9
NEDO-32686-A	"Utility Resolution Guidance for ECCS Suction Strainer Blockage".	6C
RPA 89-112	"ABB Atom Control Rod Drop Accident Analysis Methodology for Boiling Water Reactors The RAMONA-3B Computer Code", November 1989.	15.0 15.4
RPA 90-90-P-A	"BISON - A One Dimensional Dynamic Analysis Code for Boiling Water Reactors", December 1991.	15.0 16.3
UTLR-0005-P	"Post LOCA Suppression Pool Swell Analysis for ABWR Containment Design", Rev. 0, September 2009.	3B
UTLR-0007-P	"Condensation Oscillation and Chugging Load", Rev. 0, October 2010.	3B
UTLR-0010-P	"Toshiba Task Report for Supporting Analysis for Emergency Operating Information and Controls", Rev. 0.	18H
UTLR-0011	"ABWR Probabilistic Evaluations".	19D 19K
UTLR-0012	"ABWR PRA-Based Reliability".	19K

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
UTLR-0013	"ABWR Shutdown Risk Evaluation".	19K 19L 19Q 19QA 19QB
UTLR-0014	"ABWR Severe Accident Evaluations".	19E 19EA 19EE
UTLR-0015	"ABWR Risk-significant Human Actions".	19.10
UTLR-0016	"ABWR Severe Accident Analysis".	19E
UTLR-0017	"ABWR Containment Ultimate Strength".	19FA
UTLR-0019	"ABWR Fire Probabilistic Risk Assessment".	19M
WCAP-15836-P-A	"Fuel Rod Design Methods for Boiling Water Reactors - Supplement 1", April 2006.	4.2
WCAP-15942-P-A	"Fuel Assembly Mechanical Design Methodology for Boiling Water Reactors Supplement 1 to CENPD-287", March 2006.	4.2 3.9 1.3
WCAP-16081-P-A	"10x10 SVEA Fuel Critical Power Experiments and CPR Correlations: SVEA-96 Optima2", March 2005.	4.4
WCAP-16096-P-A	"Software Program Manual for Common Q Systems".	1.8
WCAP-16097-P-A	"Common Qualified Platform Topical Report", Revision 0.	1.8
WCAP-16530-NP-A	"Evaluation of Post-Accident Chemical Effects in Containment Sump Fluids to Support GSI-191", March 2008.	6C
WCAP-16606-P-A	"Supplement 2 to BISON Topical Report RPA 90-90-P-A", Revision 1, August 2009.	15.0
WCAP-16747-P	"POLCA-T: System Analysis Code with Three-Dimensional Core Model", Revision 0, March 2007.	15.0
WCAP-17058-P	"Implementation of ABWR Methodology using GOTHIC for STP 3 and 4 Containment Design Analyses", Westinghouse Electric Company LLC, June 2009.	6.2 16.3
WCAP-17065-P	"Westinghouse Secondary Containment Analysis Methodology", Revision 0.	6.2
WCAP-17079-P	"Supplement 3 to BISON Topical Report RPA 90-90-P-A SAFIR Control System Simulator", Rev. 0, October 2009.	15.0
WCAP-17116-P	"Westinghouse BWR ECCS Evaluation Model: Supplement 5 - Application to the ABWR", September 2009.	6C 6.3

Table 1.6-1 Referenced Reports (Continued)

Report No.	Title	Tier 2 Section No.
WCAP-17119-P	"Methodology for STP Units 3 and 4 ABWR Technical Specification Setpoints", Rev. 2.	7.3 8.3 16.5
WCAP-17137-P	"Westinghouse Stability Methodology for the ABWR", Rev. 0, October 2010.	15.9
WCAP-17181-P	"ABWR Reactor Internal Pump Motor-Generator Set Flywheel Integrity Criteria and Methodology", Revision 1, August 2011.	3.5
WCAP-17202-P	"Supplement 4 to BISON Topical Report RPA 90-90-P-A - Extended Qualification of BISON", June 2010.	15.0
WCAP-17203-P	"Supplement 5 to BISON Topical Report RPA 90-90-P-A - Fast Transient and ATWS Methodology", June 2010.	15.0
WCAP-17256	"STP 3 ABWR Prototype Reactor Internals Flow-Induced Vibration Assessment Program", Rev. 2, June 2011.	3.9
WCAP-17290-P	"Supplemental Information for Toshiba ABWR DCD Renewal Amendment", Rev. 0.	4.3 4A 4B 4D 6.3
WCAP-17322-P	"Reference Safety Report for Boiling Water Reactor Fuel and Core Analyses - Supplement 1 to CENPD-300-P-A," September 2010.	4.1 4.3 4.4