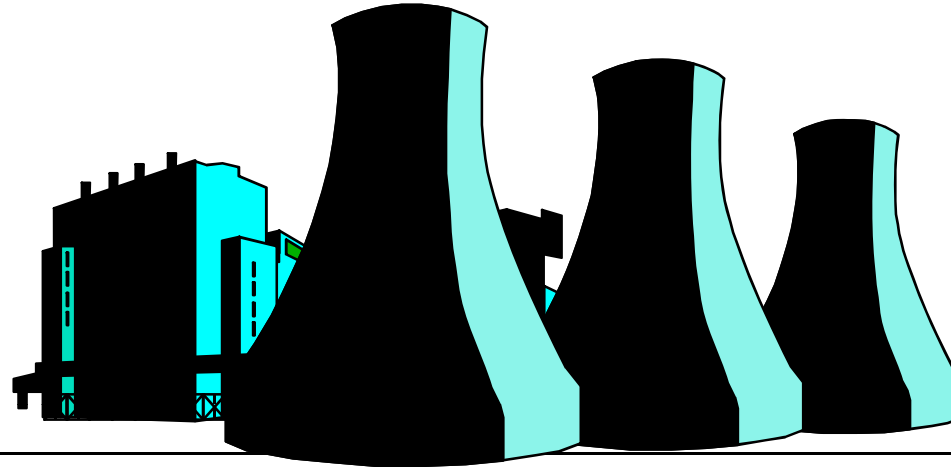




# G-105, Conducting Inspections

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**Welcome!**



# Course Logistics

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- Conclude by 4 pm on day one
- 8 am – 4 pm for days two, three and four
- Conclude by noon on day five
- Series of Group Activities & Workshops
- Sign student roster and turn in SIS by COB today
- Please silence cell phones and pagers
- Please do not touch printers, monitors, cameras, or other equipment in the room
- Please do not surf the internet during lecture
- Regulatory Course – not a Technology Course



# Course Introductions

---

- Name
- Office/Location
- Time with the NRC
- Qualification Status (G-104, P-105, MC 1245 basic, etc.)
- Primary Job and Job Desired
- Career Goals
- Education
- Nuclear experience and inspection experience
- Little known or interesting fact about yourself



# Course Components

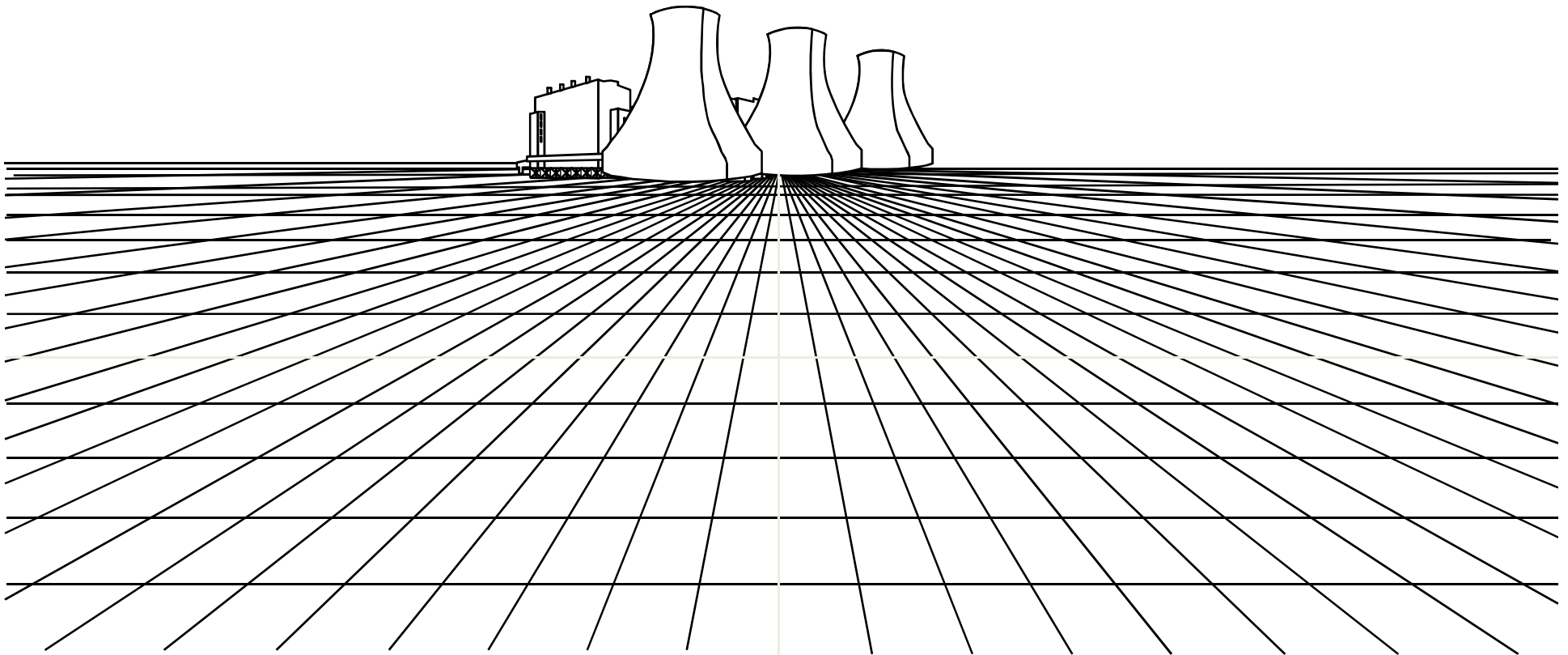
---

- Reactor Oversight Process and Inspection Concepts
- Inspection Process
  - Prepare,
  - Execute,
  - Evaluate, and
  - Document
    - Primarily Baseline Inspections
    - Workshops and Case Studies
- Significance Determination Process (SDP)



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# Reactor Oversight Process and Inspection Concepts





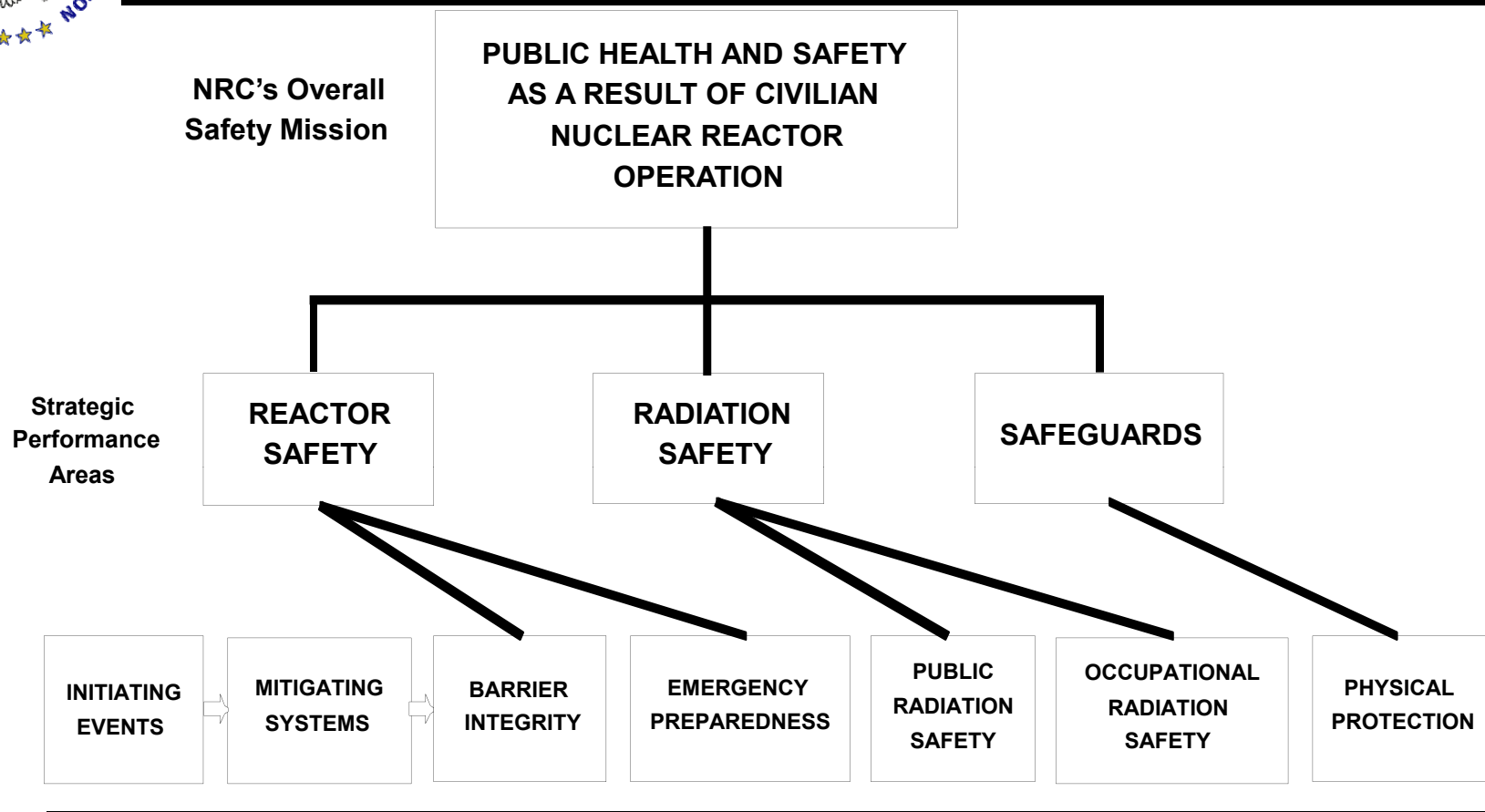
# Learning Objectives

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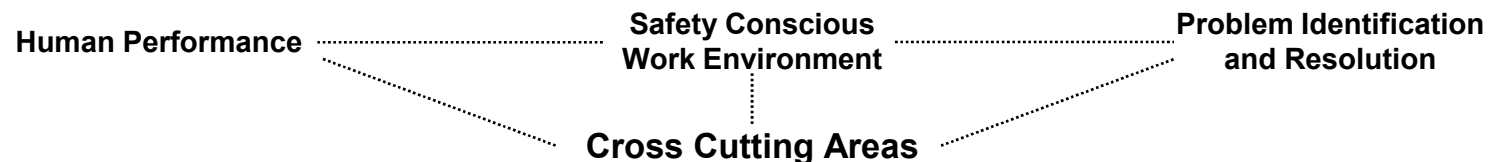
- Describe the regulatory framework of the ROP from the NRC's mission to the inspectors role.
- Describe how inspections are designed to consider cross cutting aspects associated with inspection findings.
- Describe how performance indicators are used in conjunction with inspection results.
- Describe the different ways that significance is determined in order to fully characterize findings.
- Describe the relationship between the inspection and assessment processes and the enforcement program.
- Describe the logic for increased regulatory engagement that stems from assessment results.



# Regulatory Framework



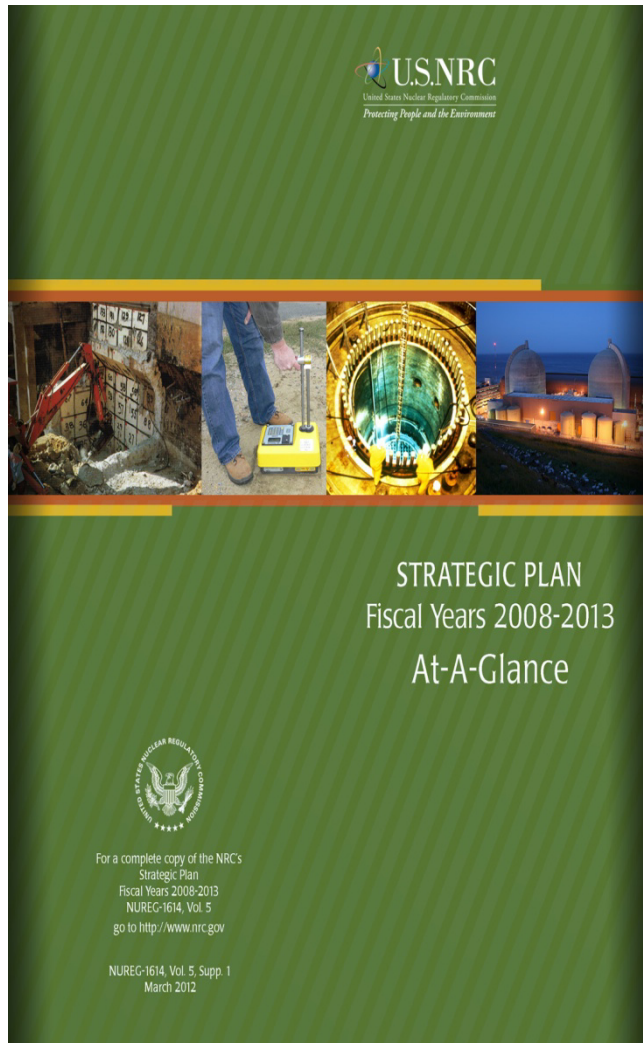
## Cornerstones



Source: MC 0305 (Operating Reactor Assessment Program ) dated 7/6/11 and  
MC 0308 (ROP Basis Document ) dated 11/8/07



# Strategic Plan



The Regulatory Framework is analogous to the Strategic Plan

- All ROP activities should relate directly to fulfilling the mission.

- If they don't, why do them?

➤ However, before reviewing the Strategic Plan...



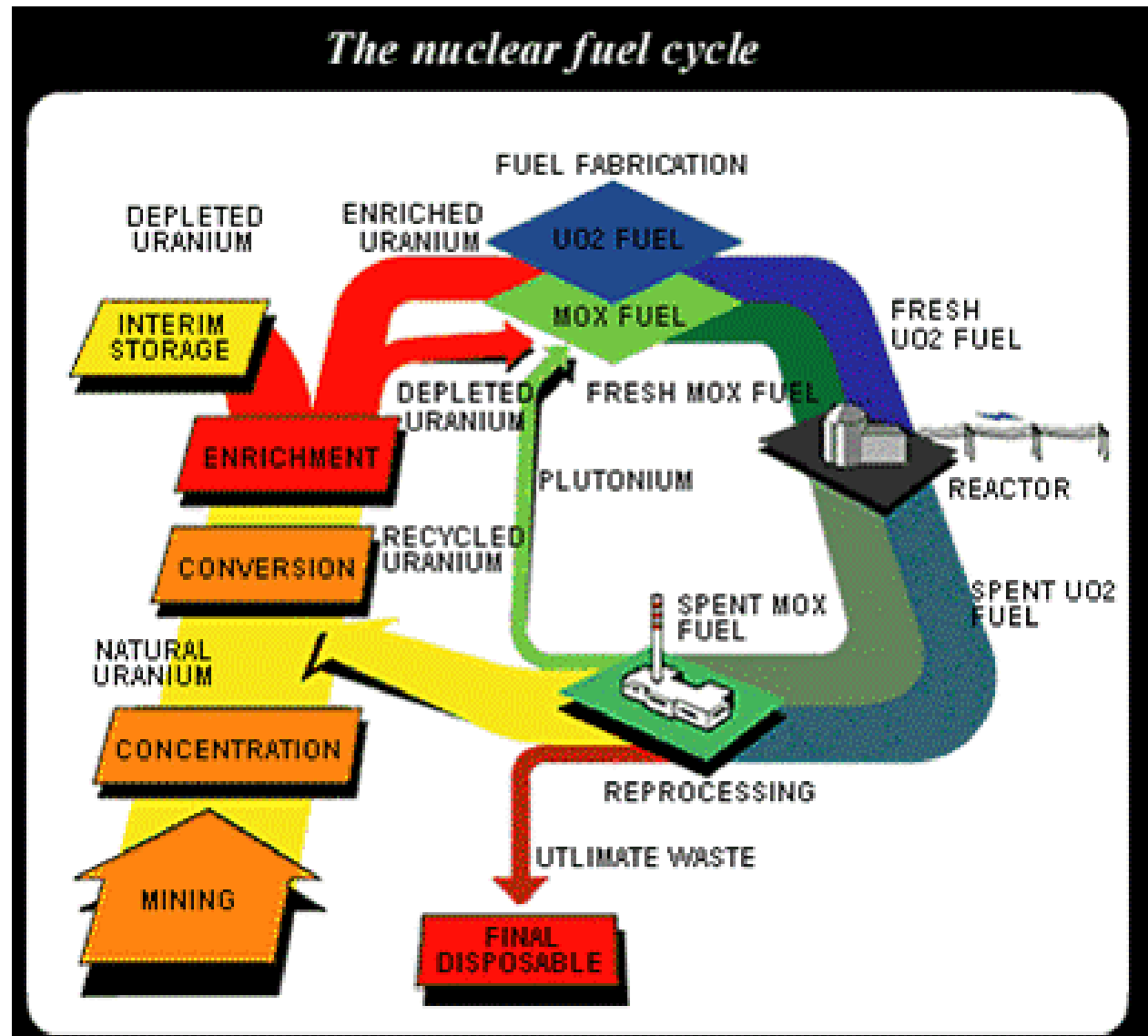


# Definitions

...let's look at some Definitions found in 10 CFR 50.2

- Byproduct material
- Source Material
- Special nuclear material

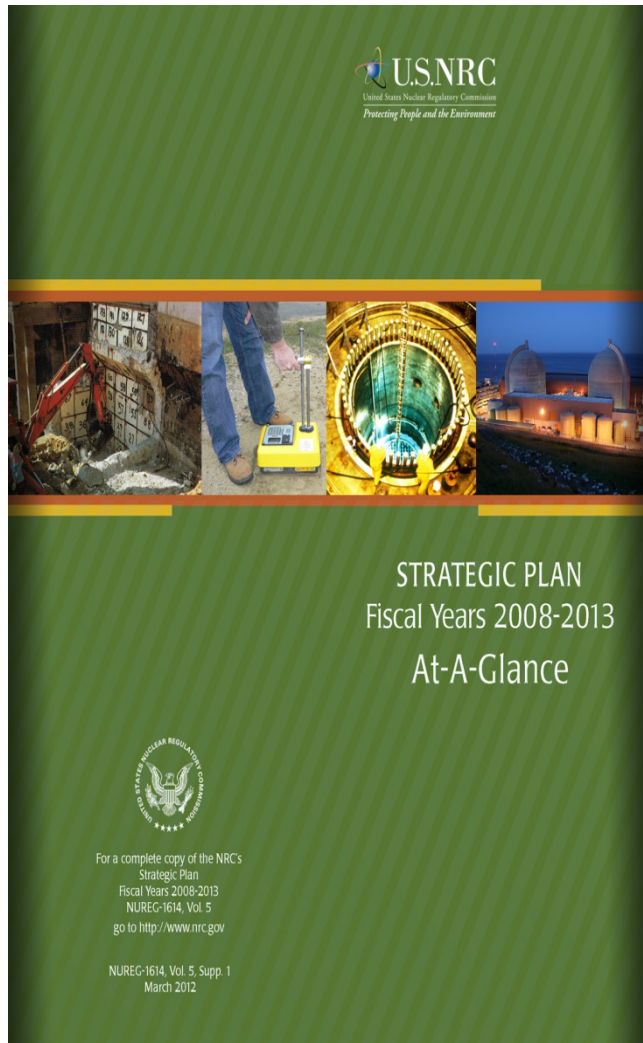
➤ Now, let's take a look at the Strategic Plan





# Strategic Plan

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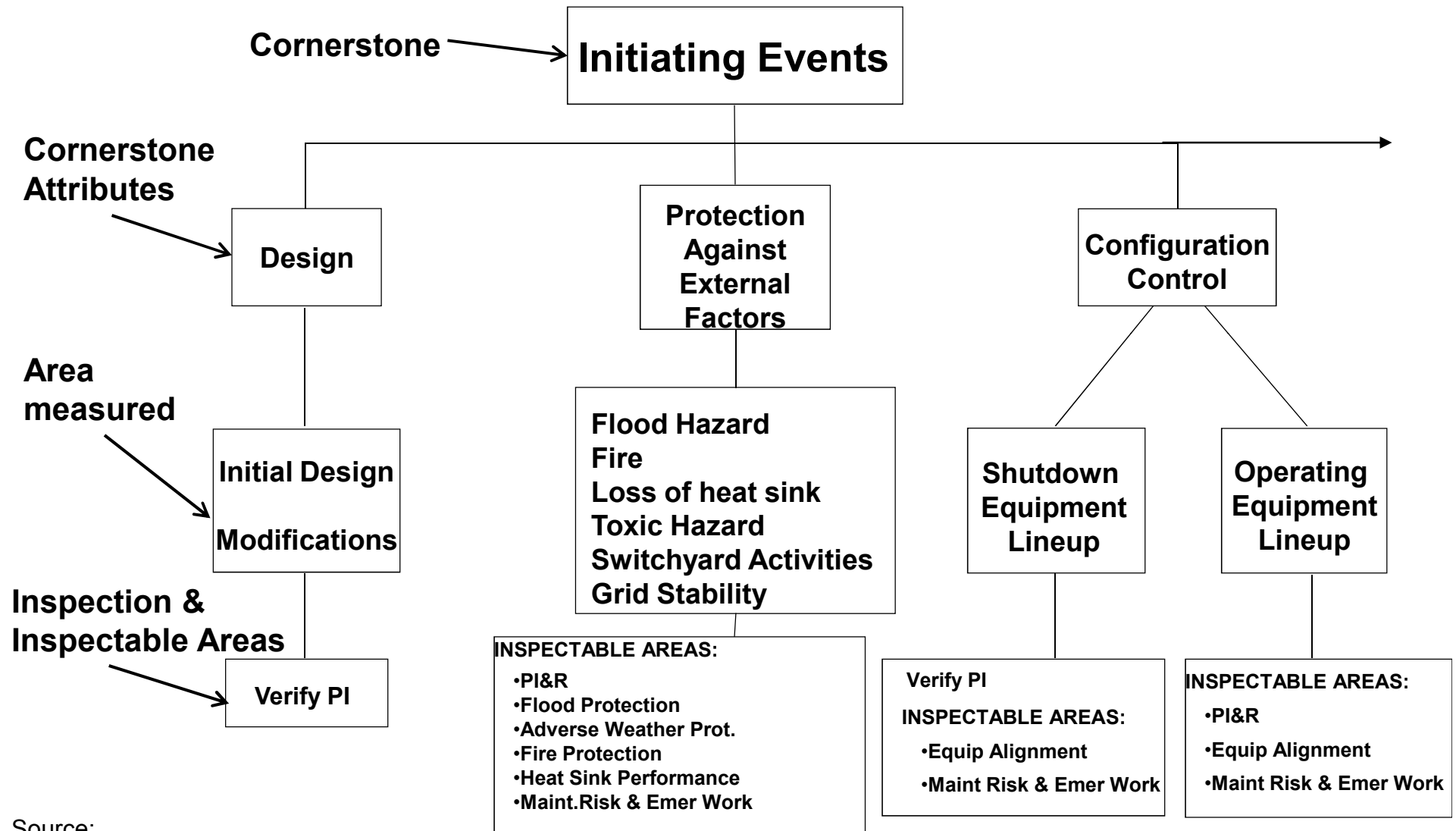


## Strategic Plan

- About the NRC
- Mission
- NRC Organizational Values
- Strategic Goals
  - Safety
  - Security
- Strategic Outcomes



# Cornerstone Detail

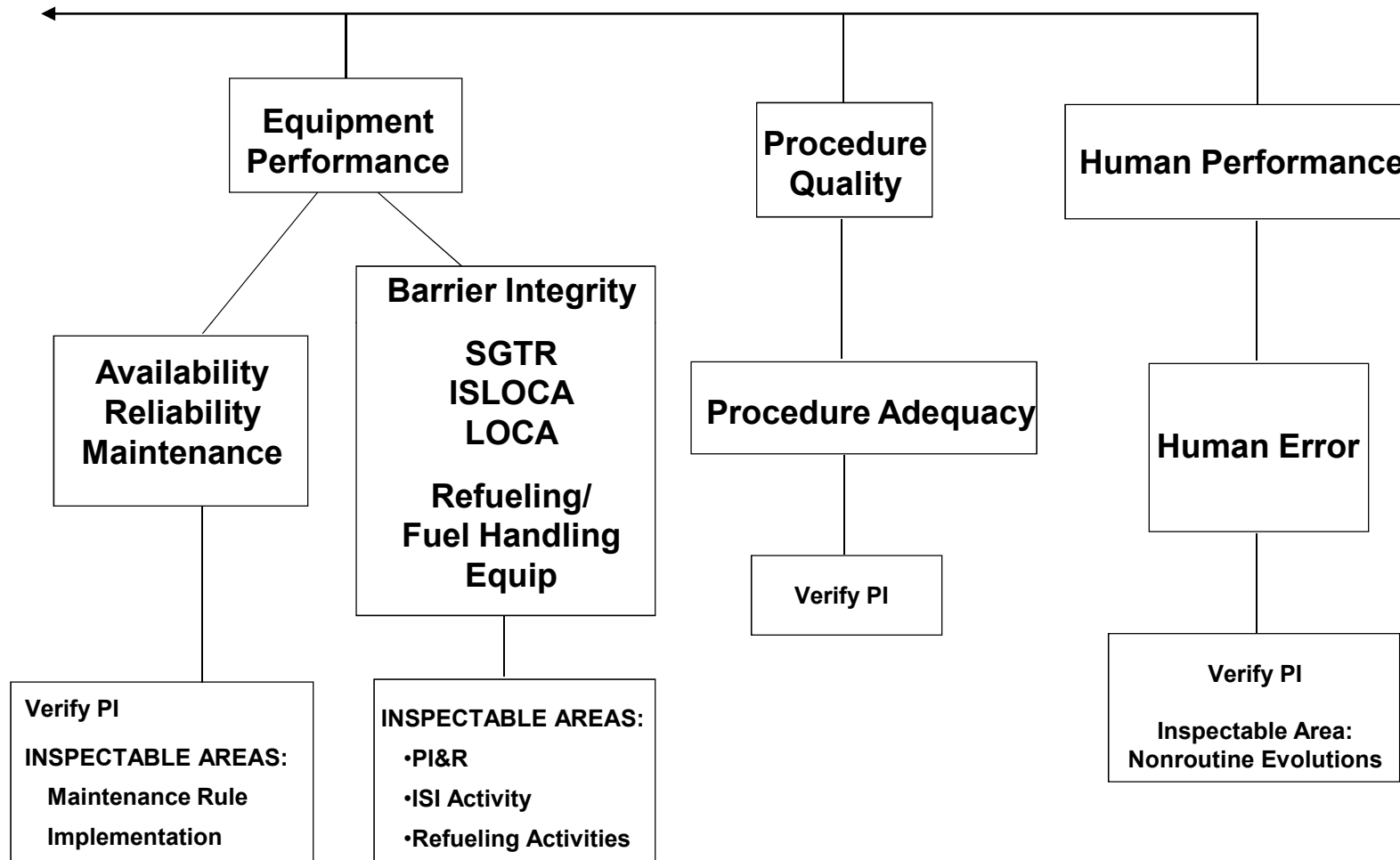


Source:

MC 2515 App A (Risk Informed Baseline Inspection Program) dated 10/28/11 and MC 0308 (ROP Basis Document ) dated 11/8/07



# Cornerstone Detail



Source: MC 2515 App A (Risk Informed Baseline Inspection Program) dated 10/28/11 and MC 0308 (ROP Basis Document ) dated 11/8/07



# Inspectable Area Details

## ATTACHMENT 1 Inspectable Area by Cornerstone

The baseline inspection program requires the inspectable areas below be reviewed at each nuclear power plant. The inspectable areas verify aspects of key attributes for each of the associated cornerstones.

Inspectable Area	Initiating Events	Mitigating Systems	Barrier Integrity	Emergency Preparedness	Occupational Radiation Safety	Public Radiation Safety	Security
Access control to radiologically significant areas					X		
Access authorization program							X
Access control							X
Adverse weather protection	X	X					
ALARA planning and controls					X		
Alert and notification system testing				X			
Component Design Bases Inspection		X					
Contingency response							X
Drill evaluation				X			
Emergency response organization augmentation testing				X			
Emergency action level and emergency plan changes				X			
Equipment alignment	X	X	X				
Equipment performance, testing, maintenance							X
Evaluations of changes, tests, or experiments and Permanent Plant Modifications		X	X				
Exercise evaluation				X			
Fire protection	X	X					



# Concept of Assessment

---

- In order to determine the overall health of a particular cornerstone the inspection and performance indicator results need to be considered together.
- Although each inspection result and performance indicator is evaluated for its own significance, the totality of results are assessed to determine the overall impact on facility safety.
- This approach enables a consistent qualitative measurement of mission success through each cornerstone and strategic performance area.



# Areas of Assessment

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- Inspectable Areas
- Cross-Cutting Areas
- Performance Indicators



# Inspectable Areas

## ATTACHMENT 3

### BASELINE INSPECTION PROCEDURES

IP/IA No.	Title	Frequency <sup>1</sup>
71111 Reactor Safety – Initiating Events, Mitigating Systems, Barrier Integrity		
71111.01	Adverse Weather Protection	A
	(Reserved)	
	(Reserved)	
71111.04	Equipment Alignment	Q/A
71111.05AQ	Fire Protection	Q/A
71111.05T	Fire Protection	T
71111.05TTP	Fire Protection – NFPA 805 Transition Period (Triennial)	T
71111.06	Flood Protection Measures	A
71111.07	Heat Sink Performance	A/T
71111.08	Inservice Inspection Activities	R
	(Reserved)	
	(Reserved)	
71111.11	Licensed Operator Requalification Program	Q/B
71111.12	Maintenance Effectiveness	A
71111.13	Maintenance Risk Assessment and Emergent Work Control	A
	(Reserved)	
71111.15	Operability Evaluations	A
	(Reserved)	
71111.17	Evaluations of Changes, Tests, or Experiments and Permanent Plant Modifications	T
71111.18	Plant Modifications	A
71111.19	Post –Maintenance Testing	A
71111.20	Refueling and Outage Activities	R
71111.21	Component Design Bases Inspection	T
71111.22	Surveillance Testing	A
	(Reserved)	
71114 Reactor Safety – Emergency Preparedness		
71114.01	Exercise Evaluation	B
71114.02	Alert Notification System Testing	B
71114.03	Emergency Response Organization Staffing and augmentation System	B
71114.04	Emergency Action Level and Emergency Plan Changes	A





# Inspectable Areas

---

- Defined by Inspection Procedures - all areas where there is a need to inspect a licensee's performance
  - Objectives provide focus (-01)
  - Requirements indicate the sample selection(-02)
  - Guidance assists with methodology (-03)
  - Resource Estimate delineates the number of hours (-04)
  - Completion Status delineates the minimum number of samples (-05)
  - References supply the technical standards (-06)

➤ Now let's explore each section of an IP in detail



# Inspectable Areas - **71111.04, Equipment Alignment**

“Objectives” section provides the focus for the IP:

## 71111.04-01 INSPECTION OBJECTIVES

01.01 To verify equipment alignment and identify any discrepancies that impact the function(s) of the system and, therefore, potentially increase risk.

01.02 To verify that the licensee has properly identified and resolved equipment alignment problems that could cause initiating events or impact the availability and functional capability of mitigating system or barrier.



# Inspectable Areas - **71111.04, Equipment Alignment**

**“Inspection Requirements” section specifies what satisfies the IP.**

1. Inspectors will perform approximately three partial walkdown inspections each calendar quarter to verify the operability of a redundant or backup system/train or a remaining operable system/train with a high risk significance for the current plant configuration (considering out-of-service (OOS), inoperable, or degraded condition);
2. or a risk-significant system/train that was recently realigned following an extended system outage, maintenance, modification, or testing; or a risk-significant single-train system.

This inspection activity will be performed during both shutdown and operating conditions to support the initiating events (IE), mitigating system (MS) and barrier integrity (BI) cornerstones.

3. In addition, inspectors will perform one complete walkdown every 6 months to support only the MS cornerstone.



# **Inspectable Areas - 71111.04, Equipment Alignment**

---

Guidance assists with inspection methodology:

- General Guidance
- Specific Guidance



# Inspectable Areas - **71111.04, Equipment Alignment**

“Resource Estimate” section provides a time estimate to complete the Inspection Procedure.

## 71111.04-04 RESOURCE ESTIMATE

The annual resource expenditure for this inspection procedure is estimated to be 68 to 92 hours to conduct partial and full system walkdowns at a site regardless of the number of reactor units at that site.



# Inspectable Areas - **71111.04, Equipment Alignment**

“Completion Status” section provides the minimum number of samples for an IP

## 71111.04-05 COMPLETION STATUS

Inspection of the minimum sample size will constitute completion of this procedure in the Reactor Programs Systems (RPS). That minimum sample size will consist of 14 samples representing 12 partial system walkdowns and 2 complete system walkdowns in a year.



# Inspectable Areas - **71111.04, Equipment Alignment**

“References” section provides the Technical Standards for the IP.

## 71111.04-06 REFERENCES

Inspection Procedure 71152, “Identification and Resolution of Problems”

Inspection Procedure 71111.13, “Maintenance Risk Assessments and Emergent Work Control”



# Inspectable Areas - Other References

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- Similar IPs (e.g., IP 71111.19 and 71111.22)
- Old Inspection Reports - what did previous inspectors review?
- Colleagues (in your office, SRA, DRS, Resident Staff)
- Lesson Plans
- OE/OpE
- NRC Generic Communications (e.g., INs, Bulletins, Generic Letters, RISs, 10 CFR 21 Reports, etc.)
- SERs, SOERs
- Vendor Information (e.g., Tech Manuals, Supplemental Info Letters (SILs), Technical Topics, etc.)
- Licensee Documents (e.g., Design Basis Documents, Surveillance Procedure, Work Orders, etc.)





# Inspectable Areas - Other References

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## ROP Digital City

- NRC Inspector Field Observation Best Practices
- Inspector Communities
- ROP PIM Inspection Findings
- Inspection Report Search
- Inspector Newsletter
  - Inspection Good Practices Checklist
  - Inspection Tips/Checklist
  - SRI Inspector Advice

Keep in touch with other inspectors/colleagues - the inspection world is much larger than just your plant(s)



# Inspectable Areas – (misc.)

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- IP 71111 (Reactor Safety - Initiating Events, Mitigating Systems, Barrier Integrity)
  - Lists baseline inspection attachments
  - Discusses general inspection guidance
- Inspectable Area Status Sheets (1 Unit, 2 Unit and 3 Unit sheets)
  - Used to track baseline inspection status
  - Not required by ROP guidance documents, but they are another tool



# Cross Cutting Areas

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## Cross-Cutting Areas

- Fundamental performance attributes that extend across all of the ROP cornerstones of safety.

These areas are:

- Human Performance (HU),
- Problem Identification and Resolution (PI&R), and
- Safety Conscious Work Environment (SCWE).



# Cross Cutting Areas

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## Cross-Cutting Area Components

A component that is directly related to one of the cross-cutting areas.

The cross-cutting area components in alphabetical order are:

- Corrective Action Program;
- Decision-Making;
- Environment for Raising Concerns;
- Operating Experience;
- Preventing, Detecting, and Mitigating Perceptions of Retaliation;
- Resources;
- Self and Independent Assessments;
- Work Control; and
- Work Practices.



# Cross Cutting Areas

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## Cross-Cutting Aspects

A performance characteristic of a finding that is the most significant causal factor of the performance deficiency.



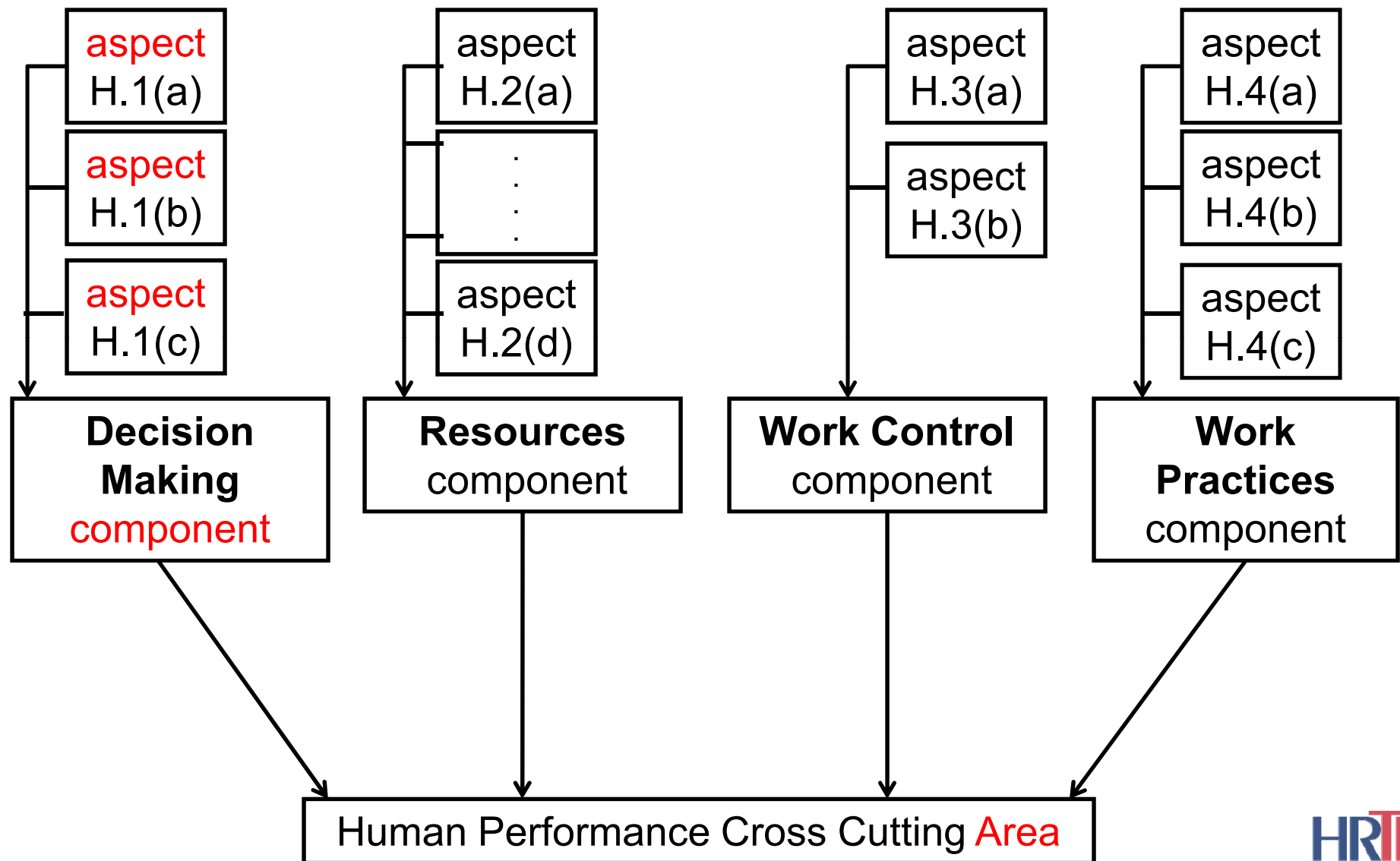
# Cross Cutting Areas

Cross Cutting Area	Human Performance	Problem Identification and Resolution	Safety Conscious Work Environment	Other Components
Cross Cutting Area Component	<ul style="list-style-type: none"> <li>▪Decision making</li> <li>▪Resources</li> <li>▪Work control</li> <li>▪Work practices</li> </ul>	<ul style="list-style-type: none"> <li>▪Operating experience</li> <li>▪Self and independent assessment</li> <li>▪Corrective action program</li> </ul>	<ul style="list-style-type: none"> <li>▪Environment for raising concerns</li> <li>▪Preventing, detecting and mitigating perceptions of retaliation</li> </ul>	<ul style="list-style-type: none"> <li>▪Safety policies</li> <li>▪Accountability</li> <li>▪Organizational change management</li> <li>▪Continuous learning environment</li> </ul>
Where Evaluated	<ul style="list-style-type: none"> <li>▪Baseline Inspections</li> </ul>			<ul style="list-style-type: none"> <li>▪ Plant specific supplemental</li> <li>▪ Event Response</li> <li>▪ Generic Safety</li> </ul>
How Evaluated	<p>A cross-cutting aspect of an inspection finding should be discussed in the report details if the inspector determines that the cross-cutting aspect of the finding was a significant contributor to the performance deficiency and if the cross-cutting aspect is indicative of current licensee performance.</p>			<p>Along with cross cutting area components, these are evaluated more directly in some non-baseline inspections.</p>

Source: MC 0310 (Components Within the Cross-Cutting Areas) dated 10/28/11



# Cross Cutting Aspects





# Cross Cutting Aspects

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## Human Performance Cross Cutting Area

<u>Work Practices</u> <input type="checkbox"/> Personnel work practices support human performance. Specifically (as applicable):	
H.4(a)	The licensee communicates human error prevention techniques, such as holding pre-job briefings, self and peer checking, and proper documentation of activities. These techniques are used commensurate with the risk of the assigned task, such that work activities are performed safely. Personnel are fit for duty. In addition, personnel do not proceed in the face of uncertainty or unexpected circumstances.
H.4(b)	The licensee defines and effectively communicates expectations regarding procedural compliance and personnel following procedures.
H.4(c)	The licensee ensures supervisory and management oversight of work activities, including contractors, such that nuclear safety is supported.





# Performance Indicators

---

Cornerstone	Indicator
Initiating Events	<ul style="list-style-type: none"><li>▪ Unplanned Scrams per 7000 critical hrs</li><li>▪ Unplanned Scrams with Complications per 7000 critical hrs</li><li>▪ Unplanned Power Changes per 7000 critical hrs</li></ul>
Mitigating Systems	<ul style="list-style-type: none"><li>▪ Safety System Functional Failures</li><li>▪ Mitigating System Performance Index</li></ul>
Barrier Integrity	<ul style="list-style-type: none"><li>▪ Reactor Coolant System Activity</li><li>▪ Reactor Coolant System Leakage</li></ul>
Emergency Preparedness	<ul style="list-style-type: none"><li>▪ Drill/Exercise Performance</li><li>▪ Emergency Response Organization Drill Participation</li><li>▪ Alert and Notification System Reliability</li></ul>
Occupational Radiation Safety	<ul style="list-style-type: none"><li>▪ Occupational Exposure Control Effectiveness</li></ul>
Public Radiation Safety	<ul style="list-style-type: none"><li>▪ Radiological Effluent Technical Specifications (RETS)/Offsite Dose Calculation Manual (ODCM)</li><li>▪ Radiological Effluent Occurrences</li></ul>
Physical Protection	<ul style="list-style-type: none"><li>▪ Not Publicly Available</li></ul>



# Performance Indicators – Initiating Events Cornerstone

---

The objective of this cornerstone is to limit the frequency of those events that upset plant stability and challenge critical safety functions, during shutdown as well as power operations.

- Unplanned Scrams per 7000 Critical Hrs - This PI calculates the number of unplanned automatic and manual scrams during the previous four quarters while critical. The number of scrams is weighted by the ratio of 7000 hours to the total number of hours of critical operation in the past four quarters.
- Unplanned Scrams with Complications - This PI calculates the number of unplanned automatic and manual scrams while critical during the previous four quarters that were complicated because of additional operator actions or unavailable equipment, as described in NEI 99-02.
- Unplanned Power Changes per 7000 Critical Hrs - This PI calculates the number of unplanned changes and fluctuations in reactor power of greater than 20 percent. This number is weighted by the ratio of 7000 hours to the total number of hours of critical operation in the past four quarters.



# Performance Indicators

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UNITED STATES NUCLEAR REGULATORY COMMISSION
Protecting People and the Environment

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## ROP Performance Indicators Summary

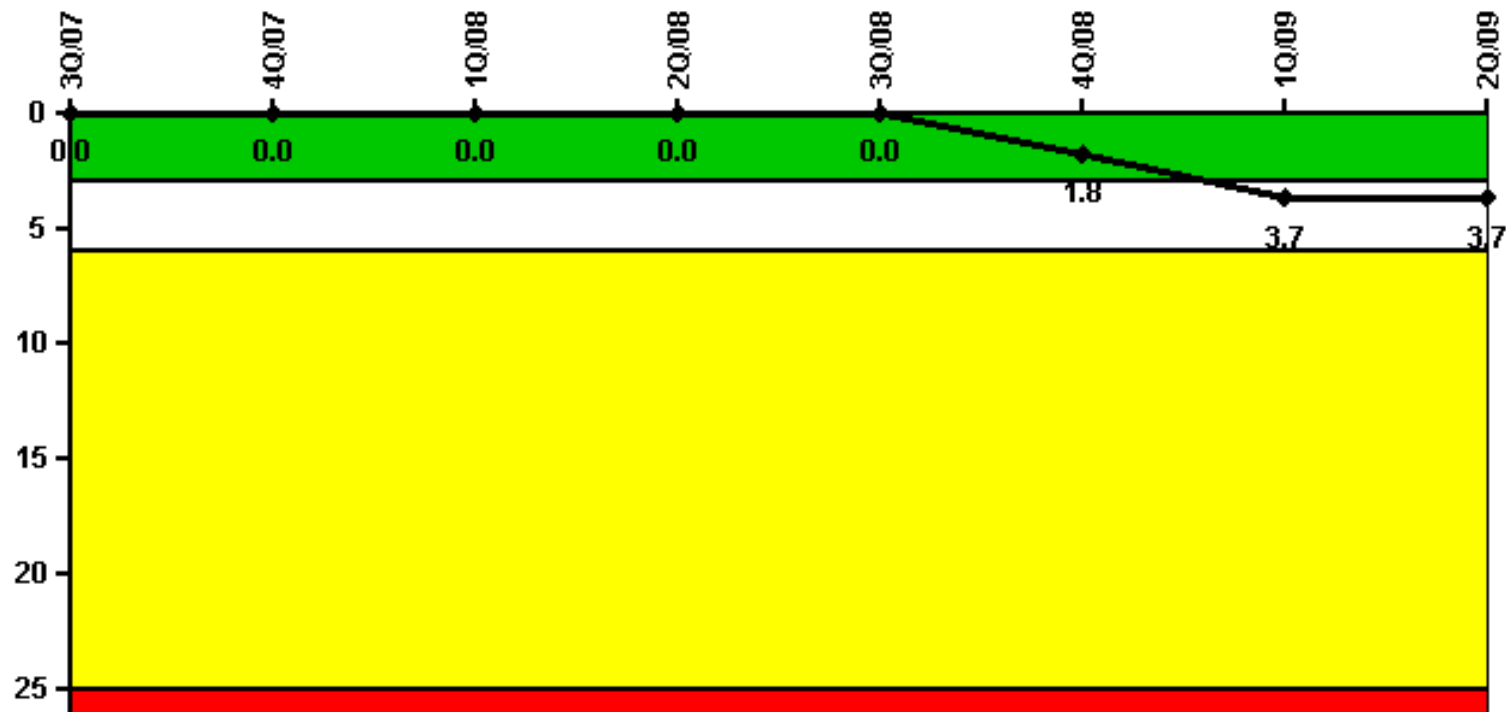
*Security information not publicly available.*

Plants	IE 01	IE 03	IE 04	MS 05	MS 06	MS 07	MS 08	MS 09	MS 10	BI 01	BI 02	EP 01	EP 02	EP 03	OR 01	PR 01
Arkansas Nuclear 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Arkansas Nuclear 2	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Beaver Valley 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Beaver Valley 2	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Braidwood 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Braidwood 2	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Browns Ferry 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Browns Ferry 2	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Browns Ferry 3	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Brunswick 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Brunswick 2	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G
Byron 1	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G	G



# Performance Indicators – Initiating Events Cornerstone

Unplanned Scrams per 7000 Critical Hrs



Thresholds: White > 3.0 Yellow > 6.0 Red > 25.0



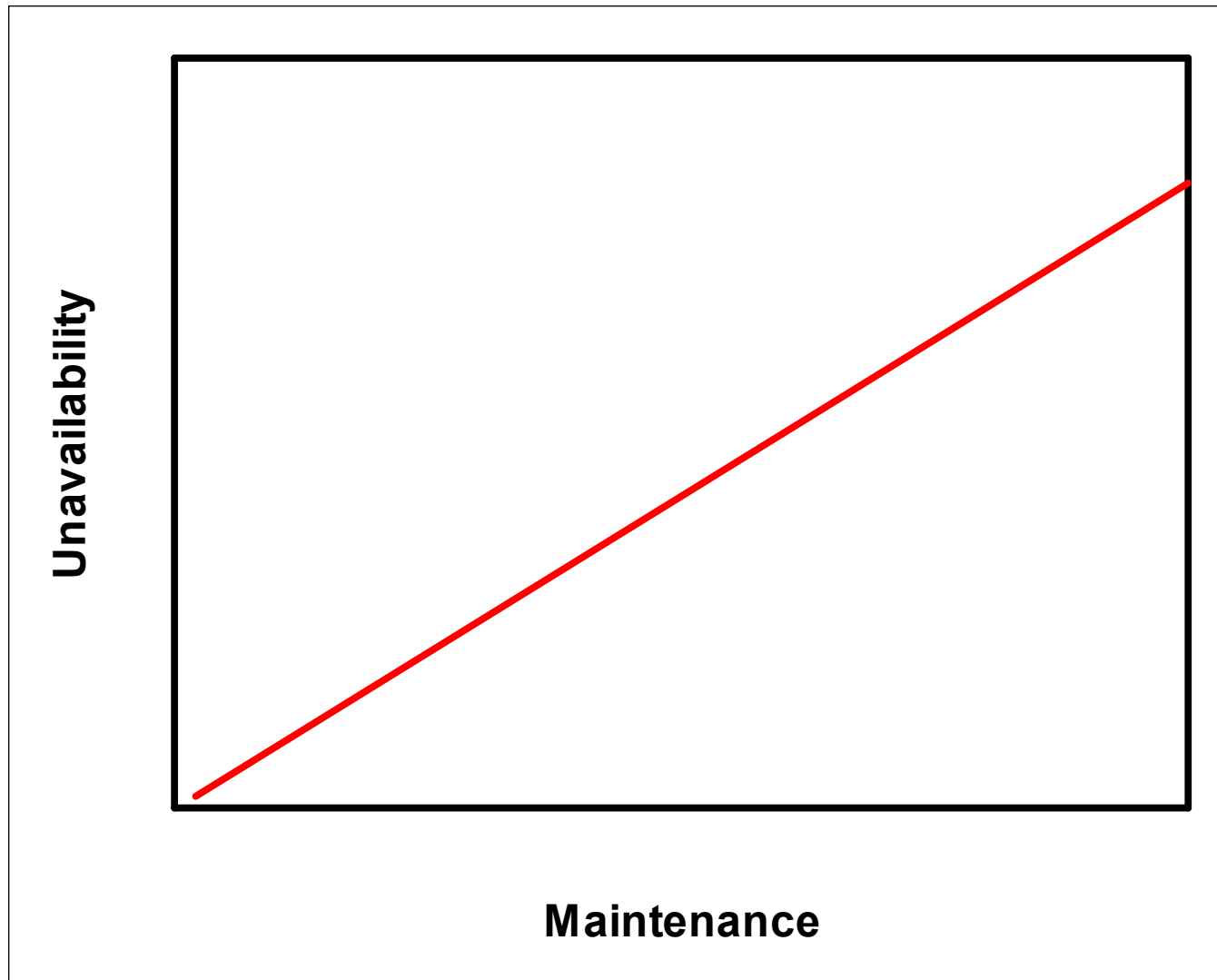
# Mitigating System Performance Index

Monitored Systems	
<u>BWR</u>	<u>PWR</u>
High pressure coolant injection/core spray	HPSI (high pressure safety injection)
RCIC or isolation condenser	AFW (auxiliary or emergency feedwater)
RHR (residual heat removal)	RHR (may include containment spray)
EAC (emergency AC power)	EAC
Cooling Water Support Systems	Cooling Water Support Systems



## ***More maintenance increases Unavailability...***

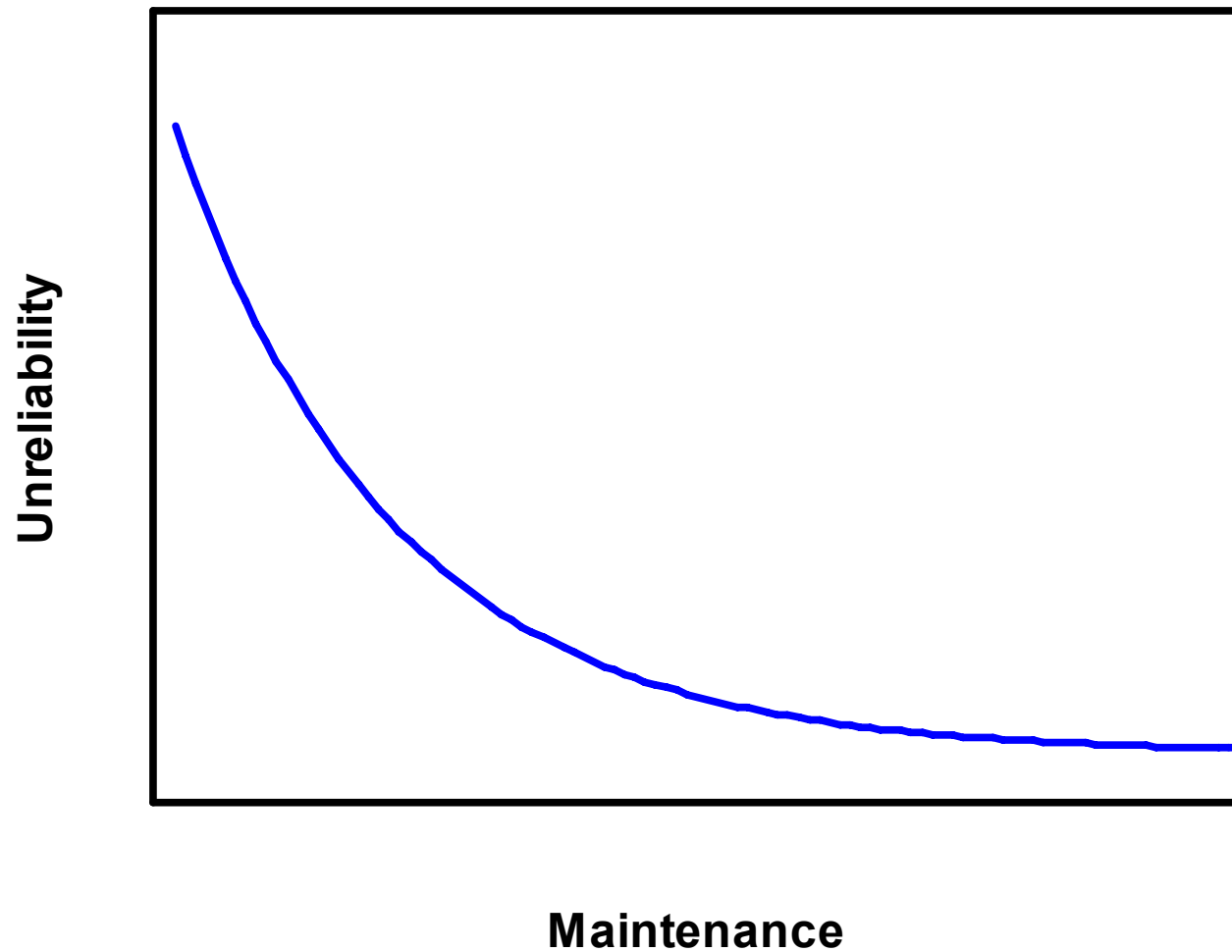
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***...but it decreases Unreliability...***

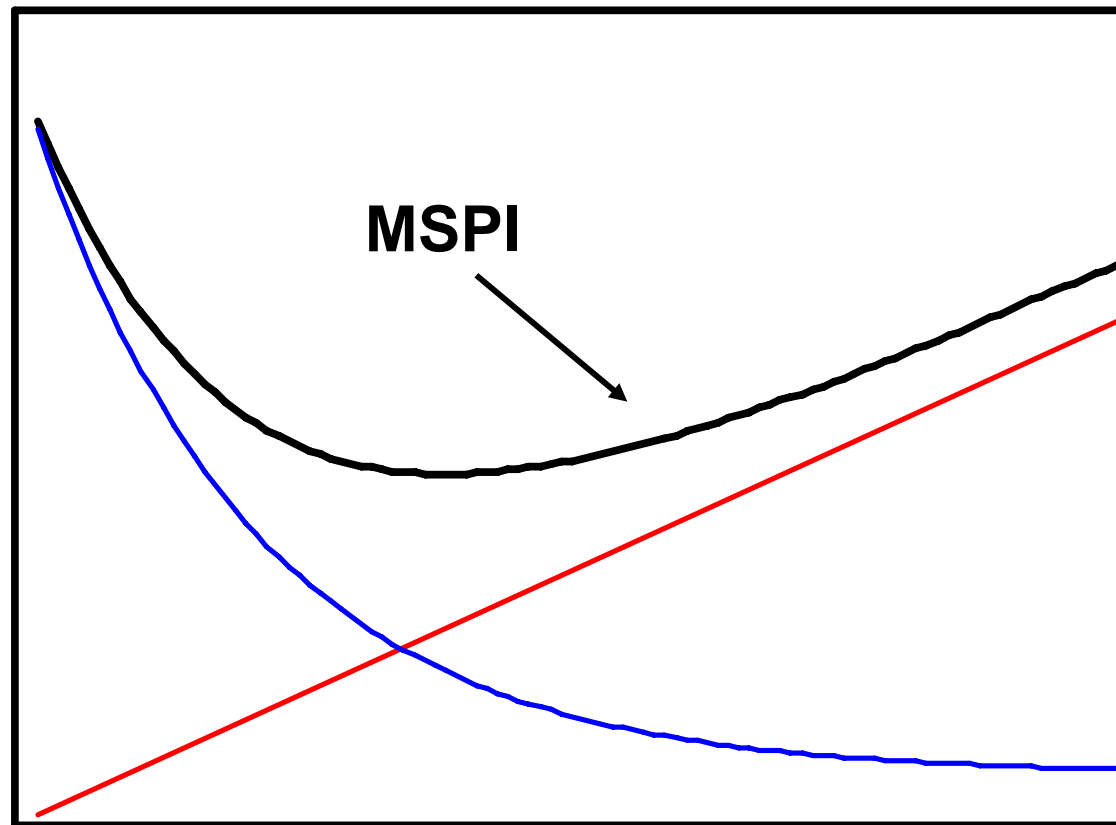
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*...and there is an optimum!*

Unavailability + Unreliability

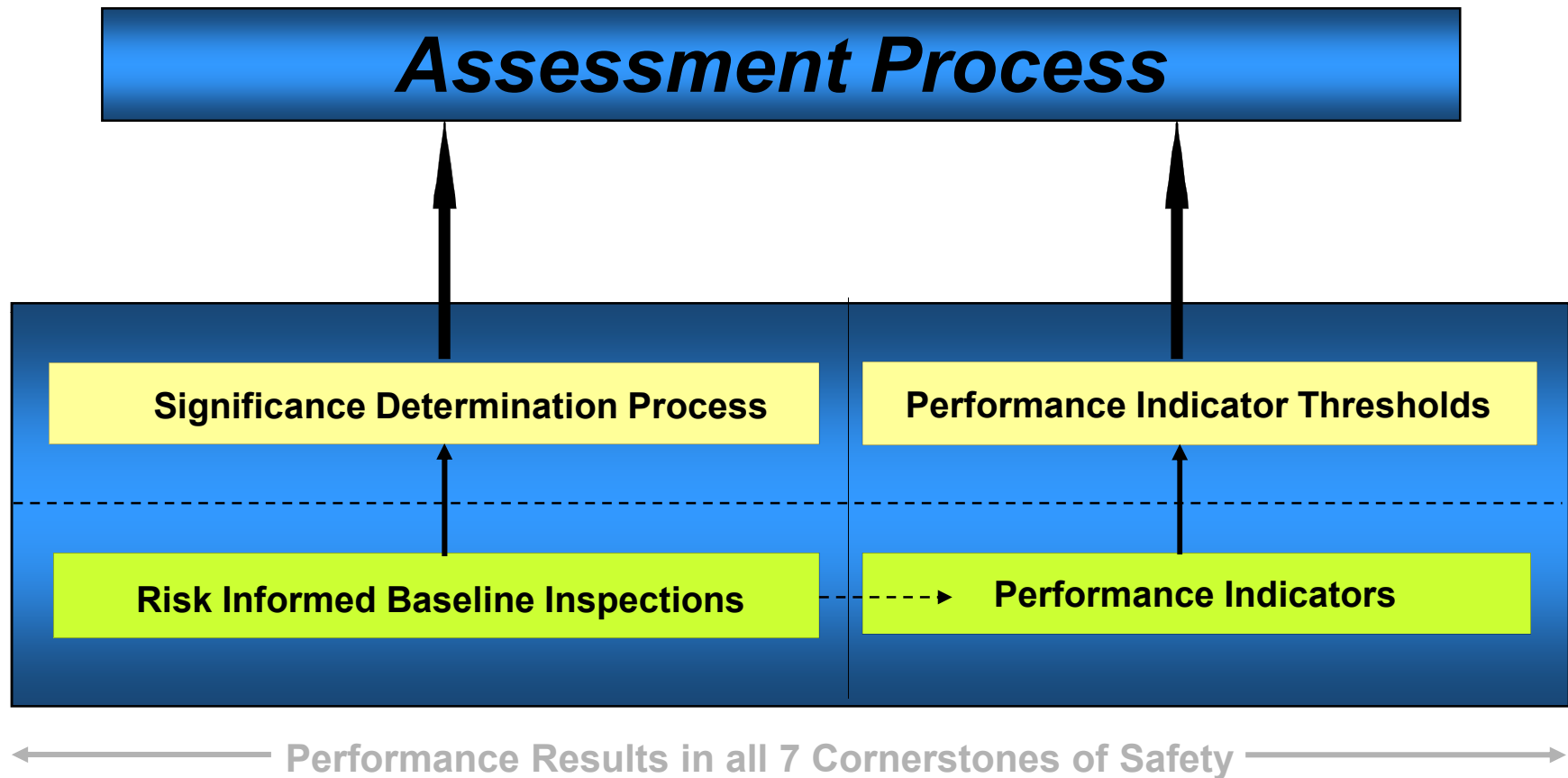


Maintenance





# Baseline Assessment Input





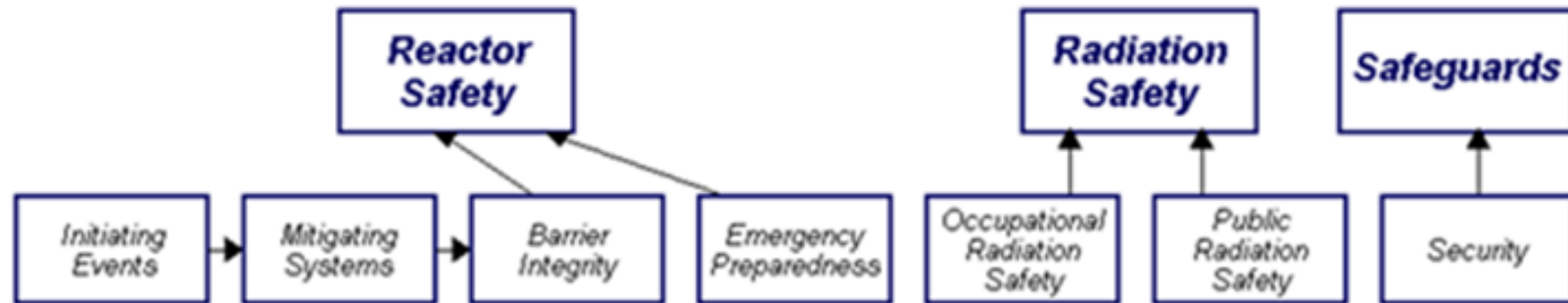
# Concept of Assessment

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# Concept of Assessment – Inspection Findings



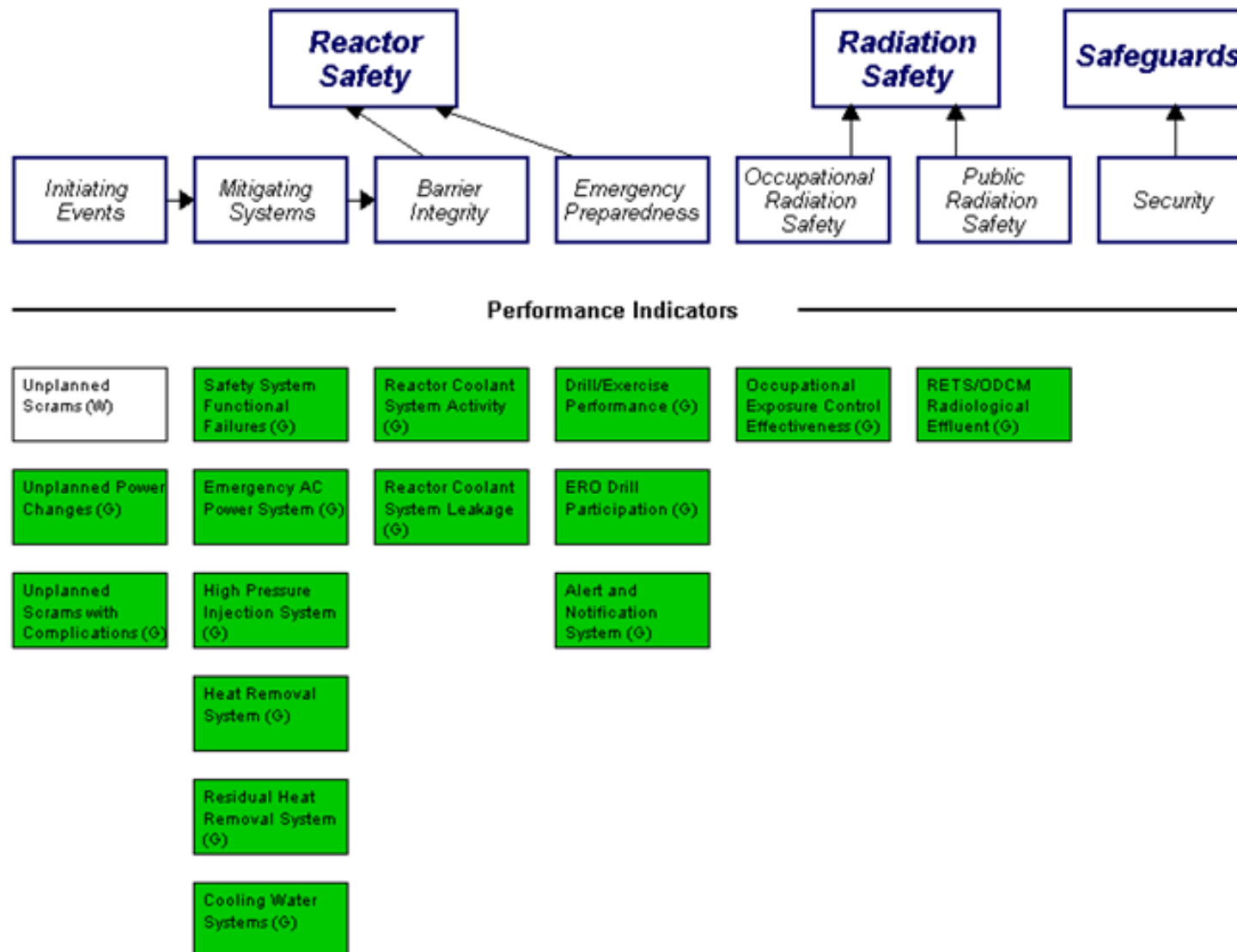
## Most Significant Inspection Findings

No findings this quarter	<b>G</b>	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter
<b>G</b>	<b>Y (1)</b>	No findings this quarter	No findings this quarter	<b>G</b>	No findings this quarter
No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter	No findings this quarter
No findings this quarter	<b>G</b>	<b>G</b>	No findings this quarter	<b>G</b>	No findings this quarter

Miscellaneous  
findings



# Concept of Assessment – Performance Indicators





# ROP Process Activities

Figure 2: Assessment Activities

Level of Review	Frequency/Timing	Participants (* indicates chairperson)	Desired Outcome	Communication
Continuous	Continuous	SRI, RI, regional inspectors, SRAs, DIRS	Performance awareness	None required; Notify licensee by an assessment follow-up letter <u>only</u> if thresholds crossed
Quarterly	Once per quarter; Five weeks after end of quarter	Division of Reactor Projects (DRP): BC*, PE, SRI, RI; DIRS	Input/verify PI/PIM data; Detect early trends	Update data set; notify licensee by an assessment follow-up letter <u>only</u> if thresholds crossed
Mid-Cycle	At mid-cycle; Seven weeks after end of second quarter	Divisions of Reactor Safety (DRS) or DRP DD*, DRP and DRS BCs, DIRS	Detect trends; Plan inspection	Mid-cycle letter with an inspection plan of approximately 15 months
End-of-Cycle	At end-of-cycle; Seven weeks after end of assessment cycle	DRS or DRP DD, RAs*, BCs, principal inspectors, SRAs, DIRS, HQ offices as appropriate	Assessment of plant performance, oversight and coordination of regional actions	Annual assessment letter with an inspection plan of approximately 15 months
End-of-Cycle Summary Meeting	Scheduled within one week after the completion of the last regional end-of-cycle review	NRR OD, RAs, DIRS, OE, OI, other HQ offices as appropriate	Summarize results of the end-of-cycle review	Information to be discussed at Agency Action Review Meeting.
Agency Action Review Meeting	Annually; Several weeks after issuance of the annual assessment letters	EDO*, NRR OD, RAs, DRS/DRP DDs, DIRS, OE, OI, other HQ offices as appropriate	Review of the appropriateness of <b>NRC</b> actions	Commission briefing, followed by public meetings with individual licensees to discuss assessment results, as appropriate



# Agency Action Matrix

Figure 1: Reactor Oversight Process Action Matrix

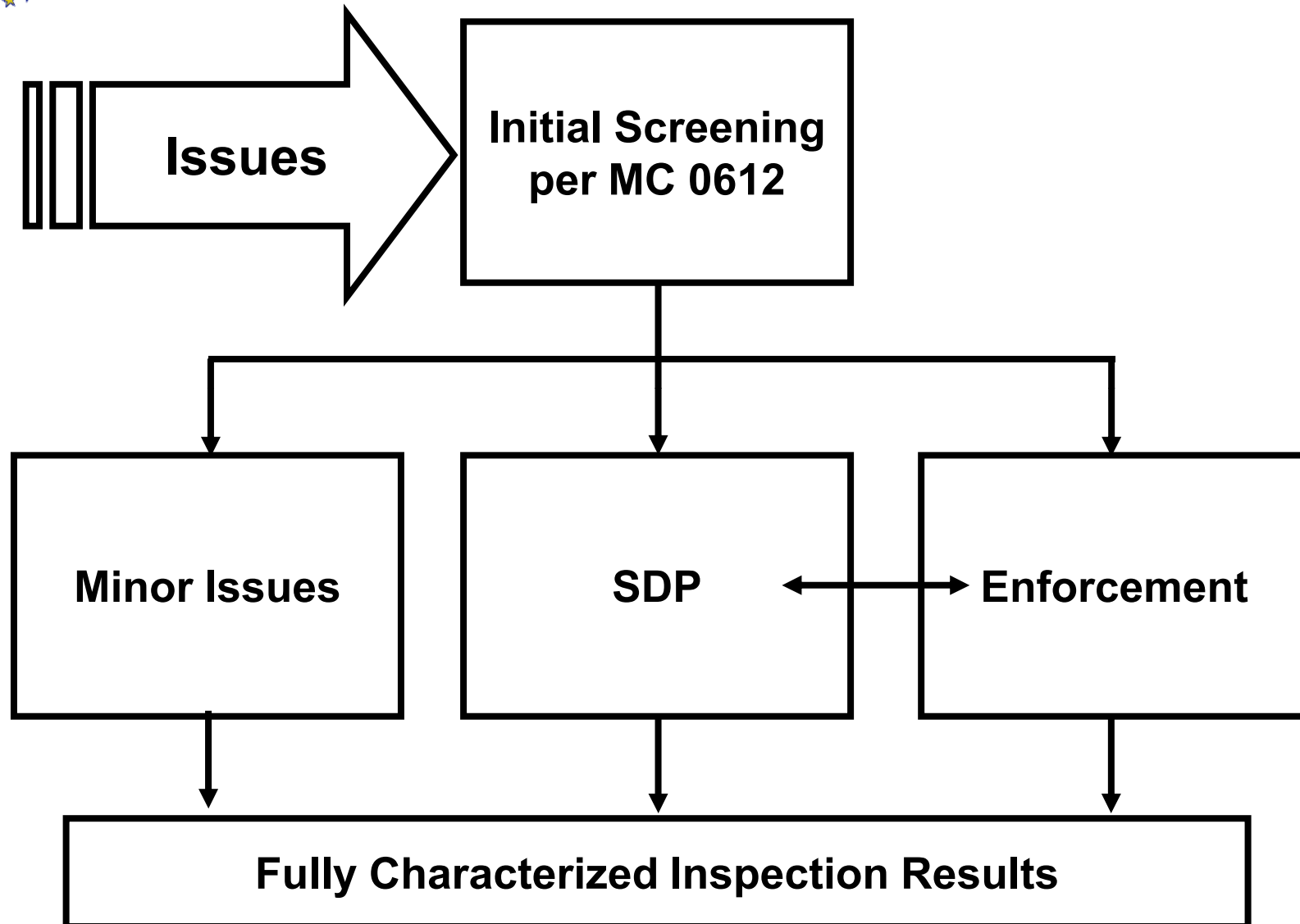
		Licensee Response Column (Column 1)	Regulatory Response Column (Column 2)	Degraded Cornerstone Column (Column 3)	Multiple/Repetitive Degraded Cornerstone Column (Column 4)	Unacceptable Performance Column (Column 5)	IMC 0350 Process <sup>1</sup>
RESULTS		All assessment inputs (performance indicators and inspection findings) green; Cornerstone objectives fully met	One <b>white</b> input, or Two white inputs (in different cornerstones) in a strategic performance area; Cornerstone objectives met <b>with minimal degradation in safety performance</b>	One degraded cornerstone (2 white inputs or 1 yellow input), or Any 3 white inputs in a strategic performance area; Cornerstone objectives met with moderate degradation in safety performance	Repetitive degraded cornerstone, Multiple degraded cornerstones, Multiple yellow inputs, or One red input; Cornerstone objectives met with longstanding issues or significant degradation in safety performance	Overall unacceptable performance; Plants not permitted to operate within this band; Unacceptable margin to safety	Plants in a shutdown condition with performance problems <b>are</b> placed in the IMC 0350 process
RESPONSE	Regulatory Performance Meeting	None	Branch Chief or Division Director meets with licensee	Regional Administrator or designee meets with senior licensee management.	EDO/DEDO or designee meets with senior licensee management	EDO/DEDO or designee meets with senior licensee management	RA/EDO or designee meets with senior licensee management
	Licensee Action	Licensee corrective action	Licensee root cause evaluation and corrective action with NRC oversight	Licensee cumulative root cause evaluation with NRC oversight	Licensee performance improvement plan with NRC oversight		Licensee performance improvement & restart plan with NRC oversight
	NRC Inspection	Risk-informed baseline inspection program	Baseline and supplemental inspection (IP 95001)	Baseline and supplemental inspection (IP 95002)	Baseline and supplemental inspection (IP 95003)		Baseline and supplemental as practicable; Special inspections per restart checklist.
	Regulatory Actions <sup>2</sup>	None	Supplemental inspection only	Supplemental inspection only; Plant discussed at AARM if conditions met	10 CFR 2.204 DFI; 10 CFR 50.54(f) letter; CAL/Order; Plant Discussed at AARM	Order to modify, suspend, or revoke license; Plant discussed at AARM	CAL/Order requiring NRC approval for restart; Plant discussed at AARM
COMMUNICATION	Assessment Letters	Branch Chief or Division Director reviews and signs assessment <b>letter</b> w/ inspection plan	Division Director reviews/signs assessment <b>letter</b> w/ inspection plan	Regional Administrator reviews/signs assessment <b>letter</b> w/ inspection plan	Regional Administrator reviews/signs assessment <b>letter</b> w/ inspection plan		N/A. RA or 0350 Panel Chairman review/ sign 0350-related correspondence
	Annual Involvement of Public Stakeholders	Various public stakeholder options involving the senior resident inspector or Branch Chief	Various public stakeholder options involving the BC or DD	Regional Administrator or designee discusses performance with senior licensee management	EDO/DEDO or designee discuss performance with senior licensee management		N/A. 0350 Panel Chairman conducts periodic public status meetings
	Commission Involvement	None	None	Possible Commission meeting if licensee remains for 3 years	Commission meeting with senior licensee management within 6 months.	Commission meeting with senior licensee management	Commission meetings as requested; Restart approval in some cases.
INCREASING SAFETY SIGNIFICANCE →							

<sup>1</sup> The IMC 0350 Process column is included for illustrative purposes only and is not necessarily representative of the worst level of licensee performance. Plants in the IMC 0350 oversight process: considered outside the auspices of the ROP Action Matrix. See IMC 0350, "Oversight of Reactor Facilities in a Shutdown Condition due to Significant Performance and/or Operational Concerns," information.

<sup>2</sup> Other than the CAL, the regulatory actions for plants in the Multiple/Repetitive Degraded Cornerstone and IMC 0350 columns are not mandatory **NRC** actions. However, the regional office should consider each of these regulatory actions when significant new information regarding licensee performance becomes available.

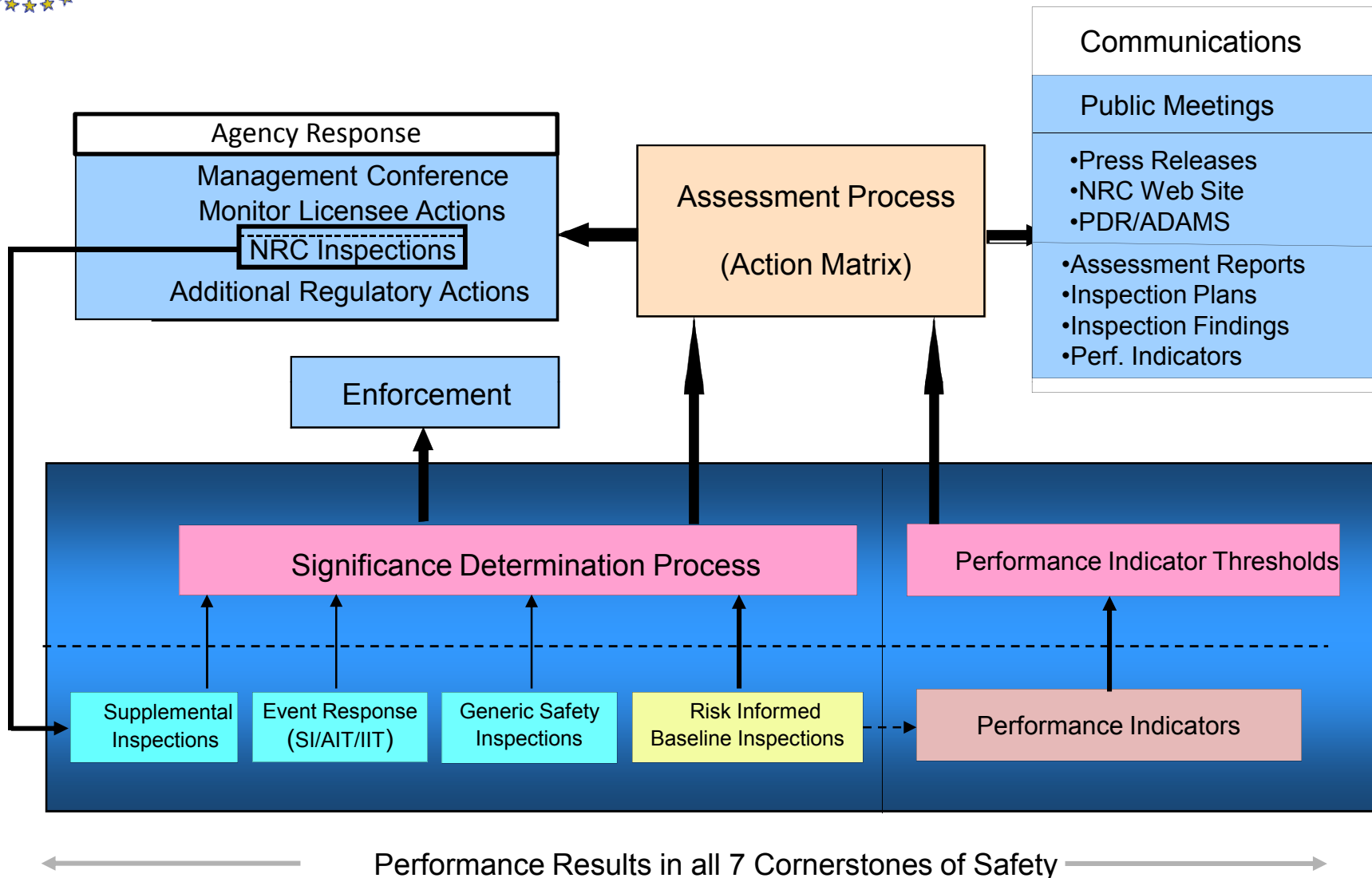


# Characterizing Inspection Results





# ROP Summary



Source: MC 0305 (Operating Reactor Assessment Program ) dated 7/6/11  
and MC 0308 (Reactor Oversight Program Basis Document ) dated 11/8/07





# Review - Learning Objectives

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- Describe the regulatory framework of the ROP from the NRC's mission to the inspectors role.
- Describe how inspections are designed to consider cross cutting aspects associated with inspection findings.
- Describe how performance indicators are used in conjunction with inspection results.
- Describe the different ways that significance is determined in order to fully characterize findings.
- Describe the relationship between the inspection and assessment processes and the enforcement program.
- Describe the logic for increased regulatory engagement that stems from assessment results.