



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 28, 2012

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT 2 - ISSUANCE OF AMENDMENT RE:
SNUBBER SURVEILLANCE REQUIREMENTS (TAC NO. ME7221)

Dear Mr. Heacock:

The Commission has issued the enclosed Amendment No. 310 to Renewed Facility Operating License No. DPR-65 for Millstone Power Station Unit 2, in response to your application dated September 21, 2011, as supplemented by letter dated February 24, 2012.

The proposed amendment would revise Technical Specification surveillance requirements for snubbers to conform to the revised inservice inspection program, move the specific surveillance requirements of TS 3/4.7.8, "Snubbers," to the "Snubber Examination, Testing, and Service Life Monitoring Program," add a reference to the program in the administrative controls section, and make administrative changes to TS 3/4.7.8.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, reading "James Kim", is positioned above the typed name.

James Kim, Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures:

1. Amendment No. 310 to DPR-65
2. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 310
Renewed License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated September 21, 2011, as supplemented by letter dated February 24, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

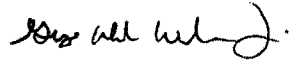
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 310, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License
and Technical Specifications

Date of Issuance: June 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 310

RENEWED FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
Page 3

Insert
Page 3

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove
XVIII
3/4 7-21
3/4 7-22
3/4 7-22a
3/4 7-22b
3/4 7-32
6-33

Insert
XVIII
3/4 7-21
3/4 7-22
3/4 7-22a
3/4 7-22b
3/4 7-32
6-33

Connecticut, in accordance with the procedures and limitations set forth in this renewed operating license;

- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2700 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 310, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

Renewed License No. DPR-65
Amendment No. 310

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PLANT SYSTEMS

3/4.7.8 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.8 All snubbers shall be OPERABLE. The only snubbers excluded from the requirements are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.8 on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for the system.

SURVEILLANCE REQUIREMENTS

4.7.8 Each snubber shall be demonstrated OPERABLE by performance of the Snubber Examination, Testing, and Service Life Monitoring Program.

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ADMINISTRATIVE CONTROLS

6.27 CONTROL ROOM ENVELOPE HABITABILITY PROGRAM (Continued)

- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of Surveillance Requirement 4.0.2 are applicable to the frequencies for assessing CRE habitability and determining CRE unfiltered leakage as required by paragraph c.

6.28 SNUBBER EXAMINATION, TESTING, AND SERVICE LIFE MONITORING PROGRAM

This program conforms to the examination, testing, and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

- a. This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.
- b. The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (BPV) Code and the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b), subject to its limitations and modifications, and subject to Commission approval.
- c. The program shall, as allowed by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants" in lieu of Section XI of the ASME BPV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(a)(3).
- d. The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(b)(3)(v)) subject to the limitations and modifications listed therein.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 310

RENEWED FACILITY OPERATING LICENSE NO. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE POWER STATION, UNIT 2

DOCKET NOS. 50-336

1.0 INTRODUCTION

By application dated September 21, 2011 (Accession No. ML11270A051), as supplemented by letter dated February 24, 2012 (ML12062A069), Dominion Nuclear Connecticut, Inc. (DNC or the licensee), submitted a request for changes to the Millstone Power Station Unit 2 (MPS2) Technical Specifications (TSs). The proposed amendment would revise the Snubber TS 3/4.7.8 to conform with planned revisions to the snubber inservice inspection (ISI) and testing program. Specifically, TS Surveillance Requirement (SR) 4.7.8 would be revised to replace the TS requirements for snubber examination, testing and service life monitoring. The TS SR would reference the inservice inspection (ISI) program requirements for snubbers, which will be located in the Administrative Controls section of TSs via an addition of a "Snubber Testing Program" under the Administrative Control Section of TS 6.28. Additionally, the associated TS 3/4.7.8 Bases section would be revised. MPS2 is currently in the fourth 10-year ISI interval. The fourth 10-year ISI interval at MPS2 began on April 1, 2010. Currently snubber examination and testing are performed in accordance with the specific requirements of TS 3/4.7.8. The proposed amendment removes the specific snubber inservice examination and testing requirements from TSs, because MPS2 is using the snubber inservice examination, testing and service life requirements as described in the American Society of Mechanical Engineers (ASME) *Code for Operation and Maintenance of Nuclear Power Plants* (OM Code), 2004 Edition and as required by Section 50.55a of Title 10 of the *Code of Federal Regulation* (10 CFR 50.55a).

The supplemental letter dated February 24, 2012, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on November 29, 2011 (76 FR 73730).

2.0 REGULATORY EVALUATION

10 CFR 50.36, "Technical specifications," the Commission established its regulatory requirements related to the content of TSs. This regulation requires that the TSs include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCO);

(3) SRs; (4) design features; and (5) administrative controls. The regulation does not specify which particular requirements need to be in the TSs for a plant.

In general, there are two classes of changes to TSs: (1) Changes needed to reflect modifications to the design basis (TSs are derived from the design basis), and (2) voluntary changes to take advantage of the evolution in policy and guidance as to the required content and preferred format of TSs over time. This amendment deals with the second class of changes. In determining the acceptability of revising TS 3/4.7.8, the NRC staff used the accumulation of generically approved guidance in NUREG-0212, Revision 2, "Standard Technical Specifications for Combustion Engineering Pressurized Water Reactors," dated Fall 1980, and NUREG-1432, Standard Technical Specifications, Combustion Engineering Plants, Revision 4.0," dated April 2012.

3.0 TECHNICAL EVALUATION

TS LCO 3.7.8 requires snubbers to be operable and provides actions for snubber inoperability. TS SR 4.7.8 establishes periodic requirements for verifying snubber operability. Specifically, LCO 3.7.8 requires that all snubbers shall be operable during MODES 1, 2, 3 and 4. In Modes 5 and 6, only snubbers located on systems required to be operable in these MODES must be operable. TS 3.7.8 requires that inoperable snubbers be replaced or repaired within 72 hours and the initiation of an engineering evaluation to determine whether a component(s) supported by the inoperable snubber(s) is capable of meeting its intended function in the specific safety system involved. TS 3.7.8 also requires that an affected safety system or affected portions of a system shall be declared inoperable and the LCO for that system shall be entered, if the requirements of TS LCO 3.7.8 cannot be met.

The existing TS SR 4.7.8 provides the detailed snubber inservice inspection program and the requirements. The licensee proposes to revise TS SR 4.7.8 to read as follows:

Each snubber shall be demonstrated OPERABLE by performance of the Snubber Examination, Testing and Service Life Monitoring Program.

The amendment would add the following description of the "Snubber Examination, Testing, and Service Life Monitoring Program," to the Administrative Control section (Section 6.0) of the TSs:

6.28 SNUBBER EXAMINATION, TESTING, AND SERVICE LIFE MONITORING PROGRAM

This program conforms to the examination, testing, and service life monitoring for dynamic restraints (snubbers) in accordance with 10 CFR 50.55a inservice inspection (ISI) requirements for supports. The program shall be in accordance with the following:

- a. This program shall meet 10 CFR 50.55a(g) ISI requirements for supports.
- b. The program shall meet the requirements for ISI of supports set forth in subsequent editions of the Code of Record and addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel (BPV) Code and

the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) that are incorporated by reference in 10 CFR 50.55a(b), subject to its limitations and modifications, and subject to Commission approval.

- c. The program shall, as allowed by 10 CFR 50.55a(b)(3)(v), meet Subsection ISTA, "General Requirements" and Subsection ISTD, "Preservice and Inservice Examination and Testing of Dynamic Restraints (Snubbers) in Light-Water Reactor Nuclear Power Plants" in lieu of Section XI of the ASME BPV Code ISI requirements for snubbers, or meet authorized alternatives pursuant to 10 CFR 50.55a(a)(3).
- d. The 120-month program updates shall be made in accordance with 10 CFR 50.55a (including 10 CFR 50.55a(b)(3)(v)) subject to the limitations and modifications listed therein.

The reference in TS SR 4.7.8, to perform snubber functional testing in the current TS ACTION 3.7.8, is replaced with reference to meet SR 4.7.8. The MPS2, SR 4.7.8, snubber augmented, ISI program to perform, and meet (a) visual inspections, (b) visual inspection acceptance criteria, (c) functional tests, (d) hydraulic snubber functional test acceptance criteria, (e) mechanical snubbers functional acceptance criteria, and (f) snubber service life monitoring, will be revised and moved to the "Snubber Testing Program." This program will be listed under TS Administrative Control 6.28. The added "Snubber Testing Program," is based on the ASME OM Code requirements, and is equivalent to the existing requirements contained in SR 4.7.8, which are based on the ASME Boiler and Pressure Vessel Code, Section XI.

The proposed changes update the snubber, augmented, ISI program content and are consistent with the requirements of 10 CFR 50.55a. The licensee has developed a detailed "Snubber Examination, Testing, and Service Life Monitoring Program," as a licensee-controlled document at Millstone Unit 2, which will be implemented in lieu of the deleted TS snubber examination and testing requirements. This snubber examination and testing program will be based on the requirements referenced in the TS Administrative Control 6.28.

The NRC staff has reviewed the requirements for "Snubber Examination, Testing, and Service Life Monitoring Program," as described in proposed new TS 6.28. The staff also reviewed the planned revised snubber program requirements as described in the licensee's supplement dated February 24, 2012. The staff finds that the program requirements are consistent with the ISI and testing requirements for snubbers as required by 10 CFR 50.55a. Therefore, the NRC staff concludes that the proposed new TS 6.28 is acceptable.

Based on the above finding that the "Snubber Examination, Testing, and Service Life Monitoring Program," is consistent with 10 CFR 50.55a, the NRC staff further finds that the examination, testing and service life monitoring for snubbers in accordance with the Snubber Testing Program, as required by SR 4.7.8, is sufficient to demonstrate whether snubbers are OPERABLE in accordance with LCO 3.7.8. Therefore, the NRC staff concludes that the proposed changes to SR 4.7.8 are consistent with the requirements in 10 CFR 50.36(c)(3) and, therefore, are acceptable.

The NRC staff finds that the proposed change to LCO 3.7.8 to revise its existing reference to "Specification 4.7.8" is administrative in nature. Therefore, the staff concludes that this change is acceptable.

Based on the above considerations, the NRC staff concludes that the proposed amendments are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes SRs. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (76 FR 73730). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Bedi

Date: June 28, 2012

June 28, 2012

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION UNIT 2 - ISSUANCE OF AMENDMENT RE:
SNUBBER SURVEILLANCE REQUIREMENTS (TAC NO. ME7221)

Dear Mr. Heacock:

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Sincerely,

/ra/

James Kim, Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures:

1. Amendment No. 310 to DPR-65
2. Safety Evaluation

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Accession No.: ML12165A220

*See memo dated June 8, 2012

OFFICE	LPL1-1/PM	LPL1-1/LA	EPTB /BC	STSB/BC	OGC NLO	LPL1-1/BC
NAME	JKim	KGoldstein	AMcMurtray*	RElliott	BMizuno	GWilson
DATE	6/14/12	6/14/12	6/8/12	6/18/12	6/26/12	6/28/12

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