

# Acronyms and Abbreviations

AC	Alternating current	EPRI	Electric Power Research Institute
ACRS	Advisory Committee on Reactor Safeguards	ESF	Engineered safeguards feature
ADS	Automatic depressurization system	ESW	Emergency service water
ADV	Atmospheric dump valve	ESWGR	Emergency switchgear
AEOD	Office for Analysis and Evaluation of Operational Data	ET	Event tree
AFW	Auxiliary feedwater	FCI	Fuel-coolant interaction
AOP	Abnormal Operating Procedure	FIVE	Fire-Induced Vulnerability Evaluation
AOT	Allowed outage time	FMEA	Failure modes and effects analysis
AOV	Air-operated valve	FSAR	Final Safety Analysis Report
APB	Accident progression bin	FT	Fault tree
APET	Accident progression event tree	F-V	Fussell-Veseley (importance)
ASEP	Accident Sequence Evaluation Program	FW	Feedwater
ASP	Accident Sequence Precursor	GE	General Electric
ATHEANA	A Technique for Human Event Analysis	GL	Generic Letter
ATWS	Anticipated transient without scram	GSI	Generic Safety Issue
BC	Boundary condition	HCLPF	High confidence, low probability of failure
BNL	Brookhaven National Laboratory	HCR	Human Cognitive Reliability
BTP	Branch Technical Position	HEP	Human error probability
BWR	Boiling water reactor	HHSI	High-head safety injection
BWROG	BWR Owners' Group	HLW	High-level waste
BWST	Borated water storage tank	HPCI	High-pressure coolant injection
CCDF	Complementary cumulative distribution function	HPCS	High-pressure core spray
CCDP	Conditional core damage probability	HPI	High-pressure injection
CCF	Common-cause failure	HPR	High-Pressure re-circulation
CCI	Core-concrete interaction	HPSI	High-pressure safety injection
CCW	Component Cooling Water	HRA	Human reliability analysis
CDF	Core damage frequency	HVAC	Heating, ventilation, and air conditioning
CDF	Cumulative Density Function	HTGR	High-Temperature Gas Reactor
CDFM	Conservative Deterministic Failure Margin	HX	Heat exchanger
CDP	Core damage probability	ICDP	Incremental core damage probability
CE	Combustion Engineering	ICCDP	Incremental conditional core damage probability
CEOG	Combustion Engineering Owners' Group	ILERP	Incremental large early release probability
CFR	Code of Federal Regulations	ICLERP	Incremental conditional large early release probability
CLB	Current licensing basis	IE	Initiating event
CRD	Control rod drive	INL	Idaho National Laboratory
CSIP	Charging/safety injection pump	INPO	Institute for Nuclear Plant Operations
CST	Condensate storage tank	IPE	Individual Plant Examination
CW	Circulating water	IPEEE	Individual Plant Examination for External Events
DBA	Design basis accident	IREP	Interim Reliability Evaluation Program
DC	Direct current	ISA	Integrated Safety Analysis
DCH	Direct containment heating	ISI	In-service inspection
DF	Decontamination factor	ISLOCA	Interfacing system loss-of-coolant accident
DFSD	Dominant functional sequence diagram	IST	In-service testing
DHR	Decay heat removal	JCO	Justification for Continued Operation
ECCS	Emergency core-cooling system	LB	Licensing basis
EDG	Emergency diesel generator	LCO	Limiting Condition for Operation
EOOS	Equipment Out of Service System	LER	Licensee Event Report
EOP	Emergency Operating Procedure	LERF	Large early release frequency
EPA	Environmental Protection Agency	LERP	Large early release probability
EPIX	Equipment performance and information exchange system	LLNL	Lawrence Livermore National Laboratory
LOCA	Loss-of-coolant accident	LLW	Low-level waste
LOOP	Loss of offsite power	LPCI	Low-pressure coolant injection
LOSP	Loss of offsite power	LPCS	Low-pressure core spray
LP/SD	Low power and shutdown	LPI	Low-pressure injection
		LPR	Low-pressure re-circulation

LPSI	Low-pressure safety injection	RSS	Reactor Safety Study
LPZ	Low population zone	RVC	Relief valve re-close
LWR	Light water reactor	RWST	Refueling water storage tank
MAAP	Modular Accident Analysis Program	S/D	Shutdown
MACCS	MELCOR Accident Consequence Code System	SAR	Safety Analysis Report
		SBO	Station blackout
MCS	Minimal cut set	SDC	Shutdown cooling
MDP	Motor-driven pump	SDP	Significance Determination Process
MGL	Multiple Greek letter	SER	Safety Evaluation Report (Staff Evaluation Report for IPE/IPEEE)
MOV	Motor-operated valve		
MSIV	Main steam isolation valve	SG	Steam generator
MSP	Maintenance and Surveillance Program	SGTR	Steam generator tube rupture
MSPI	Mitigating System Performance Index	SHARP	Systematic Human Action Reliability Procedure
NCV	Non-cited violation		
NEI	Nuclear Energy Institute	SI	Safety injection
NMSS	Office of Nuclear Materials Safety and Safeguards	SIF	Seal injection flow
		SIT	Safety injection tank
NOED	Notice of Enforcement Discretion	SLOCA	Small loss-of-coolant accident
NPP	Nuclear Power Plant	SNL	Sandia National Laboratory
NPRDS	Nuclear Plant Reliability Data System	SPAR	Standardized Plant Analysis Risk
NRC	Nuclear Regulatory Commission	SRA	Senior Reactor Analyst
NRR	Office Nuclear Reactor Regulation	SRI	Senior Resident Inspector
NUMARC	Nuclear Management and Resources Council	SRP	Standard Review Plan
		SRV	Safety/relief valve
OOS	Out of service	SSC	Systems, structures, and components
ORAM	Outage Risk Assessment and Management	SSET	Support state event tree
ORNL	Oak Ridge National Laboratory	STG	Source term group
OSHA	Occupational Safety and Health Administration	SW	Service water
		SWGR	Switch gear
P&ID	Piping and instrumentation diagram	TBCCW	Turbine building closed cooling water
PA	Performance assessment	TDP	Turbine-driven pump
PCC	PRA Coordinating Committee	TER	Technical Evaluation Report
PCS	Power conversion system	THERP	Technique for Human Error Rate Prediction
PDS	Plant damage state		
PM	Preventive maintenance	TRC	Time reliability correlation
PORV	Power-operated relief valve	USI	Unresolved Safety Issue
POS	Plant operating state	VCT	Volume control tank
PRA	Probabilistic risk assessment	WOG	Westinghouse Owners' Group
PRT	Plant response tree		
PRV	Pressurizer power-operated relief valves		
PSA	Probabilistic safety assessment		
PSF	Performance shaping factor		
PTFG	PRA Training Focus Group		
PTS	Pressurized thermal shock		
PWR	Pressurized water reactor		
QA	Quality Assurance		
QHO	Quantitative health objective		
QRA	Quantitative risk analysis		
RAW	Risk achievement worth		
RBCCW	Reactor building closed cooling water		
RCIC	Reactor core isolation cooling		
RCP	Reactor coolant pump		
RCS	Reactor coolant system		
RES	Office of Nuclear Regulatory Research		
RG	Regulatory Guide		
RHR	Residual heat removal		
RI	Resident Inspector		
RPS	Reactor protection system		
RPV	Reactor pressure vessel		
RRW	Risk reduction worth		