



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 26, 2012

Mr. Jon A. Franke, Vice President
Crystal River Nuclear Plant (NA2C)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING – ISSUANCE OF
AMENDMENT REGARDING DEPARTURE FROM A METHOD FOR EVALUATING
THE AUXILIARY BUILDING OVERHEAD CRANE AND REVISIONS TO
ASSOCIATED COMMITMENTS (TAC NO. ME8234)

Dear Mr. Franke:

The U.S. Nuclear Regulatory Commission (Commission, NRC) has issued the enclosed Amendment No. 241 to Facility Operating License No. DPR-72 for Crystal River Unit 3 Nuclear Generating Plant in response to your license amendment request (LAR) No. 312, in your letter dated March 19, 2012. The NRC issued Amendment No. 239, Departure from a Method of Evaluation for the Auxiliary Building Overhead Crane (FHCR-5), on December 27, 2011. Amendment No. 239 was approved to be implemented within 180 days of issuance of the amendment. In LAR No. 312, Revision 0, the licensee requested additional time to complete the implementation of Amendment No. 239 from 180 days to 90 days prior to moving a spent fuel shipping cask with FHCR-5. The licensee requested extending the implementation period to allow for installation and testing of the new single failure proof FHCR-5.

A copy of the safety evaluation is enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, reading "Farideh E. Saba".

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures:

1. Amendment No. 241 to DPR-72
2. Safety Evaluation

cc w/enclosures: Distribution via ListServ



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FLORIDA POWER CORPORATION

CITY OF ALACHUA

CITY OF BUSHNELL

CITY OF GAINESVILLE

CITY OF KISSIMMEE

CITY OF LEESBURG

CITY OF NEW SMYRNA BEACH AND UTILITIES COMMISSION

CITY OF NEW SMYRNA BEACH

CITY OF OCALA

ORLANDO UTILITIES COMMISSION AND CITY OF ORLANDO

SEMINOLE ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 241
License No. DPR-72

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees), dated March 19, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

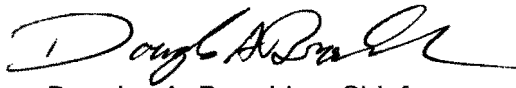
2. Accordingly, the license is amended by approving changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 241 , are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This amendment revises the completion time for Amendment No. 239 from 180 days to "Implementation shall be completed 90 days prior to moving a spent fuel shipping cask with FHCR-5." This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Douglas A. Broaddus, Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License

Date of Issuance: June 26, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 241

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following page of Facility Operating License No. DPR-72 with the attached revised page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

Remove

4

Insert

4

of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

2.C.(1) Maximum Power Level

Florida Power Corporation is authorized to operate the facility at a steady state reactor core power level not in excess of 2609 Megawatts (100 percent of rated core power level).

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 241, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

The Surveillance Requirements contained in the Appendix A Technical Specifications and listed below are not required to be performed immediately upon implementation of Amendment 149. The Surveillance Requirements shall be successfully demonstrated prior to the time and condition specified below for each.

- a) SR 3.3.8.2.b shall be successfully demonstrated prior to entering MODE 4 on the first plant start-up following Refuel Outage 9.
- b) SR 3.3.11.2, Function 2, shall be successfully demonstrated no later than 31 days following the implementation date of the ITS.
- c) SR 3.3.17.1, Functions 1, 2, 6, 10, 14, & 17 shall be successfully demonstrated no later than 31 days following the implementation date of the ITS.
- d) SR 3.3.17.2, Function 10 shall be successfully demonstrated prior to entering MODE 3 on the first plant start-up following Refuel Outage 9.
- e) SR 3.6.1.2 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- f) SR 3.7.12.2 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- g) SR 3.8.1.10 shall be successfully demonstrated prior to entering MODE 2 on the first plant start-up following Refuel Outage 9.
- h) SR 3.8.3.3 shall be successfully demonstrated prior to entering MODE 4 on the first plant start-up following Refuel Outage 9.



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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 241 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated March 19, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12081A279), Florida Power Corporation (FPC, the licensee) submitted a license amendment request (LAR) 312 for the Crystal River Unit 3 Nuclear Generating Plant (CR-3). The U.S. Nuclear Regulatory Commission (Commission, NRC) issued Amendment No. 239, Departure from a Method of Evaluation for the Auxiliary Building Overhead Crane (FHCR-5), on December 27, 2011. Amendment No. 239 was approved to be implemented within 180 days of issuance of the amendment. In LAR No. 312, Revision 0, the licensee requested additional time to complete the implementation of Amendment No. 239 from 180 days to 90 days prior to moving a spent fuel shipping cask with FHCR-5. The licensee requested extending the implementation period to allow for installation and testing of the new single failure proof FHCR-5. The licensee stated that the future replacement of the crane (FHCR-5) along with the revisions to the Final Safety Analysis Report (FSAR) will not be completed within the currently approved implementation schedule. The licensee further stated that the FSAR revisions are not appropriate until after the crane is replaced and placed in service. Therefore, the licensee requested the NRC's approval to revise the implementation schedule for Amendment No. 239.

2.0 REGULATORY ANALYSIS

In order to maintain full-core offload capacity in the spent fuel pools, CR-3 will be constructing and operating an on-site Independent Spent Fuel Storage Installation as a general licensee under the provisions of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 72, "Licensing Requirements for the Independent Storage of Spent Nuclear Fuel, High-Level Radioactive Waste, and Reactor-Related Greater Than Class C Waste," Subpart K, "General License for Storage of Spent Fuel at Power Reactor Sites." The spent fuel pools are located in the CR-3 Auxiliary Building (AB). In support of future cask loading operations, FPC is replacing the existing AB FHCR-5 with a new single failure proof crane designed in accordance with American Society of Mechanical Engineers NOG-1-2004, "Rules for Construction of Overhead and Gantry Cranes (Top Running Bridge, Multiple Girder)."

The requested revision to the implementation schedule has no safety consequences and will not have an impact on the plant's ability to store spent fuel. The proposed LAR implementation schedule change request is administrative in nature and does not change the regulatory evaluation of the NRC approved Amendment No. 239, or the applicable regulatory requirements this was based on.

3.0 TECHNICAL EVALUATION

By letter dated July 20, 2011 (ADAMS Accession No. ML11256A074), CR-3 submitted LAR 310, Revision 1, "Departure from a Method of Evaluation for the Auxiliary Building Overhead Crane, Revisions to Associated Commitments, and Response to Request for Additional Information (TAC No. ME5208)." The licensee in LAR 310 stated that it would perform analyses to qualify the new single failure proof AB overhead crane and perform calculations to qualify and upgrade the crane support structure. The NRC approved LAR 310 by letter dated December 27, 2011, and issued Amendment No. 239 (ADAMS Accession No. ML11321A165). Amendment No. 239 implementation was identified to be completed within 180 days of issuance of the amendment.

The licensee in the proposed LAR 312 stated that it is proceeding with the implementation of Amendment No. 239 and procedure changes have been issued to implement the regulatory commitment to prohibit/suspend cask handling operations when high wind conditions such as tornadoes, hurricanes, or tropical Storms are forecast. Modifications, as required, to upgrade the crane support structures are ongoing. However, the licensee stated that the installation of the new FHCR-5 and revisions to the CR-3 FSAR will not be completely implemented within 180 days of issuance of Amendment No. 239, due to unforeseen circumstances causing project scope growth.

In LAR 312, the licensee requested additional time to complete the implementation of Amendment No. 239 from 180 days to, "Implementation shall be completed 90 days prior to moving a spent fuel shipping cask with FHCR-5." The licensee stated that the requested revision to the implementation schedule will have no safety consequences because of the current load restrictions on the existing FHCR-5, and the ongoing structural upgrades to the crane support structure. The licensee added that extending the implementation period is needed to allow for installation and testing of the new single failure proof FHCR-5.

Further, the licensee stated that there is no adverse change to the plant configuration or operation as a result of a change request to the Amendment No. 239 implementation schedule. No fuel will be moved with the existing FHCR-5 and no spent fuel cask loading operations will be performed until after the new single failure proof crane is operational.

The licensee also stated that the requested revision to the implementation schedule is administrative in nature and has no safety consequences. Revising the implementation schedule will allow adequate time to complete all actions necessary for implementation. The proposed amendment implementation schedule change will not adversely impact plant or personal safety. The licensee further stated that the FSAR pages provided in LAR 310, Revision 1, dated July 20, 2011, also remain valid and unchanged.

Based on its review, the NRC staff found the licensee's request acceptable, because implementation will be completed 90 days before moving a spent fuel shipping cask with FHCR-5. No fuel will be moved with the existing FHCR-5, and no spent fuel cask loading operations will be performed until the new single-failure-proof crane is operational. The NRC staff determined that revising the implementation schedule will allow adequate time to complete all actions necessary for implementation, and it will not have an adverse effect on plant or personal safety. Further, the requested revision to the implementation schedule has no safety consequences and will not have an impact on the plant's ability to store spent fuel. In addition, the CR-3 FSAR changes provided in LAR 310, Revision 1, dated July 20, 2011, also remain valid and unchanged. Therefore, the NRC staff concluded that the licensee's proposed license amendment to complete the implementation of Amendment No. 239 from 180 days to, "Implementation shall be completed 90 days prior to moving a spent fuel shipping cask with FHCR-5" is acceptable.

4.0 STATE CONSULTATION

Based upon a letter dated May 2, 2003, from Michael N. Stephens of the Florida Department of Health, Bureau of Radiation Control, to Brenda L. Mozafari, Senior Project Manager, Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

5.0 ENVIRONMENTAL CONSIDERATIONS

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (April 17, 2012; 77 FR 22814). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Dan Hoang

Date: June 26, 2012

June 26, 2012

Mr. Jon A. Franke, Vice President
Crystal River Nuclear Plant (NA2C)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
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Sincerely,

/RA/

Farideh E. Saba, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-302

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