



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

March 13, 2012

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

**SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2 – CORRECTION RE:  
ISSUANCE OF AMENDMENTS REGARDING ADDITION OF ANALYTICAL  
METHODOLOGY TO CORE OPERATING LIMITS REPORT FOR BEST  
ESTIMATE LARGE BREAK LOSS-OF-COOLANT ACCIDENT (TAC NOS.  
ME4933 AND ME4934)**

Dear Mr. Heacock:

By letter dated February 29, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12054A168), the U.S. Nuclear Regulatory Commission (NRC) has issued Amendment Nos. 267 and 248 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units 1 and 2 (NAPS 1 and 2), respectively. The amendments change the Technical Specifications (TSs) in response to your application dated October 21, 2010, as supplemented June 23 and August 22, 2011.

These amendments add a reference to TSs 5.6.5.b, "Core Operating Limits Report (COLR)," to permit the use of the "Westinghouse Best Estimate Large Break Loss-of-Coolant Accident (BE-LBLOCA) Evaluation Methodology using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)" for the analysis of LBLOCA.

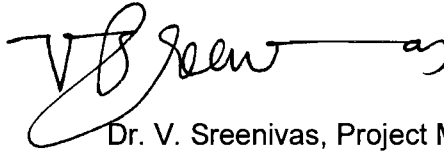
By letter dated March 7, 2012 (ADAMS Accession No. ML12068A051), Dominion identified an inadvertent typographical error on TS page 5.6-3 provided with their application and requested the NRC to re-issue the corrected TS page. Because of this error, the mark-up and the typed TS Page 5.6-3, did not match for TS 5.6.5.b. item 7. The enclosure to this letter provides a corrected Page 5.6-3 for NAPS 1 and 2.

D. Heacock

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Please contact me, if you have any questions regarding this correction letter.

Sincerely,

A handwritten signature in black ink, appearing to read 'V. Sreenivas', with a long horizontal flourish extending to the right.

Dr. V. Sreenivas, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:

1. Corrected Page 5.6-3 for NAPS, Units 1 and 2, Amendment Nos. 267 and 248.

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## 5.6 Reporting Requirements

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### 5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
1. VEP-FRD-42-A, "Reload Nuclear Design Methodology."
  2. Plant-specific adaptation of WCAP-16009-P-A, "Realistic Large Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)," as approved by NRC Safety Evaluation Report dated February 29, 2012.
  3. WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model Using the NOTRUMP Code."
  4. WCAP-10079-P-A, "NOTRUMP, A Nodal Transient Small Break and General Network Code."
  5. WCAP-12610, "VANTAGE+ FUEL ASSEMBLY-REFERENCE CORE REPORT."
  6. VEP-NE-2-A, "Statistical DNBR Evaluation Methodology."
  7. VEP-NE-1-A, "VEPCO Relaxed Power Distribution Control Methodology and Associated FQ Surveillance Technical Specifications."
  8. WCAP-8745-P-A, "Design Bases for Thermal Overpower Delta-T and Thermal Overtemperature Delta-T Trip Function."
  9. WCAP-14483-A, "Generic Methodology for Expanded Core Operating Limits Report."
  10. BAW-10227P-A, "Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel."
  11. BAW-10199P-A, "The BWU Critical Heat Flux Correlations."
  12. BAW-10170P-A, "Statistical Core Design for Mixing Vane Cores."

(continued)

D. Heacock

- 2 -

Please contact me, if you have any questions regarding this correction letter.

Sincerely,

**/RA/**

Dr. V. Sreenivas, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

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