

**(DRAFT) CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIAL PACKAGES**

1. a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
9330	5	71-9330	USA/9330/AF-96	1 OF	6

2.. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

- | | |
|---|--|
| a. ISSUED TO (<i>Name and Address</i>)
U.S. Department of Energy
Washington, DC 20585 | b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION
U.S Department of Energy consolidated application
dated June 23, 2011, as supplemented. |
|---|--|

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model No.: ATR FFSC
- (2) Description

An insulated stainless steel package for the transport of unirradiated research reactor fuel, including intact fuel elements or fuel plates. The packaging consists of (1) a body, (2) a closure lid, and (3) inner packaging internals. The approximate dimensions and weights of the package are:

Overall package outer width and height	8 inches
Overall package length	73 inches
Cavity diameter	5-3/4 inches
Cavity length	68 inches
Packaging weight (without internals)	240 pounds
Maximum package weight (including internals and contents)	290 pounds

The body is composed of two thin-walled, stainless steel shells. The outer shell is a square tube with an 8-inch cross section, a 73-inch length, and a 3/16 inch wall thickness. The inner shell is a round tube with a 6-inch diameter and a 0.120-inch wall thickness. The inner tube is wrapped with ceramic fiber thermal insulation, overlaid with a stainless steel sheet. At the bottom end, the shells are welded to a 0.88-inch thick stainless steel base plate. At the top end (closure end), the shells are welded to a 1.5-inch thick stainless steel flange.

NRC FORM 618 <small>(8-2000) 10 CFR 71</small>		U.S. NUCLEAR REGULATORY COMMISSION			
(DRAFT) CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES					
1. a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
9330	5	71-9330	USA/9330/AF-96	2 OF	6

5.(a)(2) Description (Continued)

The closure is composed of circular stainless steel plates with ceramic fiber insulation. The closure engages the top end flange by way of four bayonets that are rotated and secured by two spring pins. The closure is equipped with a handle, which may be removed during transport. The closure does not have a gasket or seal.

The package internals consist of either a Fuel Handling Enclosure for intact ATR, MIT, MURR, or RINSC fuel elements, ~~and Small Quantity Payloads~~, or a Loose Fuel Plate Basket for ATR fuel plates, ~~or a Small Quantity Payload~~. The RINSC, MIT, MURR, and Small Quantity Payload Fuel Handling Enclosures use ball lock pins and end spacers to lock closed while the ATR Fuel Handling Enclosure uses a spring plunger.

Comment [CRT1]: Reword so that the reader knows that RINSC is a small quantity. "...MURR, or RINSC elements and other Small Quantity Payloads (other than RINSC)...."

(3) Drawings

The packaging is constructed and assembled in accordance with the following Areva Federal Services LLC. or Packaging Technology, Inc., Drawing Nos.:

- | | |
|------------------------------|--|
| 60501-10, Sheets 1-5, Rev. 3 | ATR Fresh Fuel Shipping Container SAR Drawing |
| 60501-20, Rev. 1 | ATR Loose Fuel Plate Basket |
| 60501-30, Rev. 1 | ATR Fuel Handling Enclosure |
| 60501-40, Rev. 0 | MIT Fuel Handling Enclosure |
| 60501-50, Rev. 0 | MURR Fuel Handling Enclosure |
| 60501-60, Rev. 0 | RINSC Fuel Handling Enclosure |
| 60501-70, Rev. 0 | Small Quantity Payload Fuel Handling Enclosure |

(b) Contents

(1) Type and form of material

Unirradiated Mark VII Advanced Test Reactor (ATR) fuel. The ATR fuel material is composed of uranium aluminide (UAl_x). The uranium is enriched to a maximum 94 weight percent U-235; the maximum U-234 content is 1.2 weight percent; and the maximum U-236 content is 0.7 weight percent. Intact ATR fuel elements contain 19 curved fuel plates fitted within aluminum side plates, and the maximum channel thickness between fuel plates is 0.087 inch. The fuel meat thickness is a nominal 0.02 inch for all 19 plates, and the fuel meat width ranges from approximately 1.5 inches to 3.44 inches. The nominal active fuel length is approximately 48 inches. The maximum mass of U-235 per intact ATR fuel element is 1200 grams. The ATR fuel element must be contained within the ATR Fuel Handling Enclosure, as specified in 5.(a)(3).

Unirradiated MIT fuel element. The MIT fuel material is composed of uranium aluminide (UAl_x). The uranium is enriched to a maximum of 94 weight percent U-235; the maximum U-234 content is 1.2 weight percent; and the maximum U-236 content is 0.7 weight percent. Each MIT fuel element contains 15 flat fuel plates fitted within aluminum side plates and the maximum channel thickness between fuel plates is 0.090 inch. The fuel meat thickness is a

NRC FORM 618 (8-2000) 10 CFR 71		U.S. NUCLEAR REGULATORY COMMISSION				
(DRAFT) CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES						
1.	a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
	9330	5	71-9330	USA/9330/AF-96	3 OF	6

5(b)(1) Type and Form of Material (continued)

nominal 0.03 inch for all 15 plates and the fuel meat width ranges from approximately 1.98 inches to 2.17 inches. The nominal active fuel length is 22.375 inches. The maximum mass of U-235 per intact MIT fuel element is 515 grams. The MIT fuel element must be contained within the MIT Fuel Handling Enclosure, as specified in 5.(a)(3).

Unirradiated MURR fuel element. The MURR fuel material is composed of uranium aluminide (UAl_x). The uranium is enriched to a maximum of 94 weight percent U-235; the maximum U-234 content is 1.2 weight percent; and the maximum U-236 content is 0.7 weight percent. Each MURR fuel element contains 24 curved fuel plates fitted within aluminum side plates and the maximum channel thickness between fuel plates is 0.090 inch. The fuel meat thickness is a nominal 0.02 inch for all 24 plates and the fuel meat width ranges from approximately 1.71 inches to 5.72 inches. The nominal active fuel length is 24 inches. The maximum mass of U-235 per intact MURR fuel element is 785 grams. The MURR fuel element must be contained within the MURR Fuel Handling Enclosure, as specified in 5.(a)(3).

Small Quantity Payloads (RINSC fuel elements, AFIP elements, U-Mo foils, DDEs, MIT loose fuel element plates, and MURR loose fuel element plates) where the maximum mass of U-235 is 400 grams and maximum U-235 enrichment is 94%. Aluminum plates, shapes, and sheets, **and**, miscellaneous steel, or aluminum fasteners may be used as dunnage to fill gaps between the small quantity payloads and the small quantity FHE. 1/8" neoprene strips may be used between the small quantity FHE and small quantity payloads and/or between the optional aluminum dunnage and the small quantity payload. The 1/8" neoprene strips shall not be stacked in more than two layers between the small quantity payload and any interior face of the small quantity FHE.

The allowable small quantity payloads are described in the following paragraphs. U-235 loadings or U-235 enrichments exceeding the values specified in the following paragraphs are acceptable, up to the small quantity payload limits of 400 grams U-235 and 94% enrichment.

Unirradiated RINSC fuel element. The RINSC fuel material is composed of uranium silicide (U₃Si₂) dispersed in aluminum powder. The uranium is enriched to a maximum of 20 weight percent U-235; the maximum U- 234 content is 0.5 weight percent; and the maximum U-236 content is 1.0 weight percent. Each RINSC fuel element contains 22 flat fuel plates fitted within aluminum alloy side plates and the maximum channel thickness between fuel plates is 0.096 inch. The fuel meat thickness is a nominal 0.02 inch for all 22 plates. The maximum mass of U-235 per intact RINSC fuel element is 283 grams. The RINSC fuel element must be contained within the RINSC Fuel Handling Enclosure, as specified in 5-(a)(3).

AFIP fuel element. The AFIP fuel element is composed of uranium 7+/-1% weight percent molybdenum alloy **dispersed** in an aluminum-silicon matrix or uranium 10+/-1% weight percent molybdenum alloy coated with a thin zirconium interlayer. The

Comment [CRT2]: Delete "dispersed"

NRC FORM 618 (8-2000) 10 CFR 71		U.S. NUCLEAR REGULATORY COMMISSION			
(DRAFT) CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES					
1. a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
9330	5	71-9330	USA/9330/AF-96	4 OF	6

uranium is enriched to approximately 20 weight percent U-235. Each AFIP element contains 4 curved fuel plates fitted within 6061 aluminum side plates. The maximum mass of U-235 AFIP element is 365 grams. Loose plates from an AFIP fuel element are also permitted. The AFIP fuel element must be contained within the Small Quantity Payload Fuel Handling Enclosure, as specified in 5(a)(3).

5(b)(1) Type and Form of Material (continued)

U-Mo Foils. The U-Mo foils are composed of uranium 7+/-1% weight percent molybdenum alloy dispersed in an aluminum-silicon matrix or uranium 10+/-1% weight percent molybdenum alloy and may contain a zirconium coating. The uranium is enriched to a maximum of 94% U-235. The maximum mass of U-235 is 160 grams. More than one U-Mo foil type may be transported at a time. The U-Mo foils must be contained within the Small Quantity Payload Fuel Handling Enclosure, as specified in 5(a)(3).

Comment [CRT3]: Delete "dispersed"

DDEs. The DDEs are composed of uranium 7+/-1% weight percent molybdenum alloy dispersed in an aluminum-silicon matrix or uranium 10+/-1% weight percent molybdenum alloy. The uranium is enriched to a maximum of 94% U-235. The maximum mass of U-235 is 365 grams. Loose plates from a DDE are also permitted. The DDEs must be contained within the Small Quantity Payload Fuel Handling Enclosure, as specified in 5(a)(3).

Comment [CRT4]: Delete "dispersed"

MIT and MURR loose fuel element plates. MIT and MURR loose plates may either be flat or curved and may be banded or wire-tied in a bundle. The MIT and MURR loose plate payload is limited to 400 grams of U-235. The approximate mass of U-235 of each MIT fuel plate is 34.34g. The approximate mass of U-235 per each MURR fuel plate is 19 to 46 grams. The fuel plates must be contained within the Small Quantity Payload Fuel Handling Enclosure, as specified in 5(a)(3).

ATR loose fuel plates: ATR loose plates may either be flat or curved and may be banded or wire-tied in a bundle. The ATR loose plate payload is limited to 600 grams of U-235. Additional aluminum plates may be used as dunnage to fill gaps between the fuel plates and the basket payload cavity. The fuel plates must be contained within the ATR Loose Fuel Plate Basket, as specified in 5(a)(3).

(2) Maximum quantity of material per package

The maximum total weight of contents and internals, including dunnage and other secondary packaging, is 50 lbs. Radioactive contents are not to exceed a Type A quantity.

For intact ATR, MURR, RINSC, and MIT fuel elements: One fuel element.

For ATR loose fuel plates: A maximum of 600 grams U-235.

For Small Quantity Payloads: A maximum of 400 grams U-235.

**(DRAFT) CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIAL PACKAGES**

1. a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
9330	5	71-9330	USA/9330/AF-96	5 OF	6

(c) Criticality Safety Index (CSI):

For ATR, MURR, MIT fuel elements or ATR loose fuel plates: 4.0

For Small Quantity Payloads: 25

6. Fuel elements and fuel plates may be bagged or wrapped in polyethylene. The maximum weight of the polyethylene wrap shall not exceed 100 grams per package.
7. Air transport of fissile material is not authorized.
8. In addition to the requirements of 10 CFR 71 Subpart G:
 - (a) The package must be loaded and prepared for shipment in accordance with the Package Operations in Section 7 of the application.
 - (b) The package must be tested and maintained in accordance with the Acceptance Tests and Maintenance Program in Section 8 of the application.
9. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.
10. Revision No. 4 of this certificate may be used until February, 2013.
11. Expiration date: May 30, 2014.

REFERENCES

U.S. Department of Energy consolidated application dated June 23, 2011, as supplemented: August 18, 2011, and January 10, 2012.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Christine Lipa, Acting Chief
Licensing Branch
Division of Spent Fuel Storage and Transportation
Office of Nuclear Material Safety
and Safeguards

NRC FORM 618 (8-2000) 10 CFR 71		U.S. NUCLEAR REGULATORY COMMISSION			
(DRAFT) CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES					
1.	a. CERTIFICATE NUMBER	b. REVISION NUMBER	c. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE PAGES
	9330	5	71-9330	USA/9330/AF-96	6 OF 6

Date:

DRAFT