



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 22, 2012

Mr. Mano Nazar  
Executive Vice President, Nuclear and  
Chief Nuclear Officer  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

SUBJECT ST. LUCIE PLANT, UNIT 2 - REQUEST FOR ADDITIONAL INFORMATION  
REGARDING THE 2011 STEAM GENERATOR TUBE INSERVICE  
INSPECTION REPORT (TAC NO. ME7163)

Dear Mr. Nazar:

By letter dated September 20, 2011, Florida Power & Light Company submitted information summarizing the results of the spring 2011 steam generator tube inspections performed at St. Lucie, Unit No. 2, during Refueling Outage 19.

The Nuclear Regulatory Commission staff has reviewed the information provided and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). It is requested that your RAI response be provided within 30 days of the date of this letter.

Sincerely,

A handwritten signature in black ink, appearing to read "Tracy J. Orf", is positioned below the word "Sincerely,".

Tracy J. Orf, Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-389

Enclosure:  
Request for Additional Information

cc w/enclosure: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

REGARDING STEAM GENERATOR TUBE

INSERVICE INSPECTION REPORTS

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

1. Discuss the results of the tube plug inspections. Were all of the plugs inspected to confirm that all were present and that none exhibited any signs of degradation? If degradation was observed, what was observed and what corrective actions were taken?
2. Discuss the results of the feedwater ring inspection port inspections. Were all the bolts/nuts fully engaged? If not, what actions, if any, were taken to address this condition?
3. Confirm that the wear indications highlighted in the row 69 tubes are occurring at the apex of the tube where anti-vibration bar (AVB) 4 transitions into AVB 5 (i.e., the bottom of the AVB 4/5 U-bend support). Is this the only row where such wear ("apex wear") has been observed? If so, provide any insights on why this is the only row affected since it appears similar conditions exist at other locations (e.g., row 40 for AVB 3/6).
4. Discuss the results of your condition monitoring assessment. Did all tubes satisfy the structural integrity performance criteria? Was the accident induced leakage performance criteria satisfied? Were the number and severity of indications detected within your prior outage projections? If not, please discuss any corrective actions that were taken.
5. Discuss whether the tube in row 88, column 93 in SG B, with a previously reported "tube shaving signal," was inspected. In addition, confirm that the "tube shaving signal" is still no longer present.

ENCLOSURE

February 22, 2012

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/RA/

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\*By memo

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DATE	02/16/12	02/15/12	02/07/12	02/21/12	02/22/12

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