



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 15, 2012

Mr. Paul Freeman
Site Vice President
c/o Michael O'Keefe
Seabrook Station
NextEra Energy Seabrook, LLC
P.O. Box 300
Seabrook, NH 03874

SUBJECT: SEABROOK STATION, UNIT 1 - REQUEST FOR ADDITIONAL INFORMATION
REGARDING RELIEF FOR ALTERNATIVE 3AR-1, USE OF A RISK-
INFORMED, SAFETY-BASED INSERVICE INSPECTION PROGRAM
(TAC NO. ME7569)

Dear Mr. Freeman:

By letter dated November 7, 2011, NextEra Energy Seabrook, LLC (NextEra or licensee) submitted request for alternative 3AR-1 for the third 10-year inservice inspection (ISI) interval program at the Seabrook Station, Unit 1 (Seabrook). The licensee requested to implement a risk-informed, safety-based inservice inspection (RIS_B) program for the American Society of Mechanical Engineers *Boiler and Pressure Vessel Code* (ASME Code) Class 1 and Class 2 piping, Examination Categories B-F, B-J, C-F-1, and C-F-2 piping welds at Seabrook. The proposed RIS_B program is based, in part, on the ASME Code, Section XI, Code Case N-716, "Alternative Piping Classification and Examination Requirements, Section XI Division 1." The proposed alternative is applicable to Seabrook's third 10-year ISI interval which started on August 19, 2010.

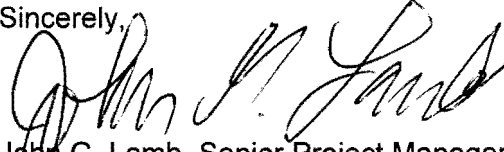
The U.S. Nuclear Regulatory Commission (NRC) has determined that additional information is required to complete its review. The NRC staff's request for additional information (RAI) is contained in the enclosed RAI. A draft of these questions was previously sent to Mr. Paul Willoughby of your staff with an opportunity to have a teleconference to ensure that the licensee understood the questions and their regulatory basis, as well as to verify that the information was not previously docketed. On February 14, 2012, Mr. Paul Willoughby of your staff agreed that NextEra Energy Seabrook, LLC would respond to the RAI within 30 days of the date of this letter. Please note that if you do not respond to the RAI within 30 days, the NRC staff may reject your request for relief under the provisions of Title 10 of the Code of Federal Regulations, Section 2.108, "Denial of application for failure to supply information."

P. Freeman

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If you have questions, you can contact me at 301-415-3100 and/or John.Lamb@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "John G. Lamb". The signature is fluid and cursive, with the first name "John" being the most prominent part.

John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION
RELIEF FOR ALTERNATIVE 3AR-1, USE OF A RISK-INFORMED,
SAFETY-BASED INSERVICE INSPECTION PROGRAM
NEXTERA ENERGY SEABROOK, LLC
SEABROOK STATION, UNIT 1
DOCKET NUMBER 50-443

1.0 SCOPE

By letter dated November 7, 2011 (Agencywide Documents Access & Management System (ADAMS) Accession Number ML11319A016), NextEra Energy Seabrook, LLC (NextEra or licensee) submitted request for alternative 3AR-1 for the third 10-year inservice inspection (ISI) interval program at the Seabrook Station, Unit 1 (Seabrook). The licensee requested to implement a risk-informed, safety-based inservice inspection (RIS_B) program for the American Society of Mechanical Engineers *Boiler and Pressure Vessel Code* (ASME Code) Class 1 and Class 2 piping, Examination Categories B-F, B-J, C-F-1, and C-F-2 piping welds at Seabrook. The proposed RIS_B program is based, in part, on the ASME Code, Section XI, Code Case N-716, "Alternative Piping Classification and Examination Requirements, Section XI Division 1." The proposed alternative is applicable to Seabrook's third 10-year ISI interval which started on August 19, 2010.

In accordance with Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a(a)(3)(i), the licensee has submitted a request for relief (3AR-1) for the third 10-year ISI program.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information submitted by the licensee, and based on this review, determined the following information is required to complete the evaluation.

2.0 REQUEST FOR ADDITIONAL INFORMATION

1. In the submittal, the licensee stated that "the Internal Flood probabilistic risk assessment (PRA) [with supporting requirements as recommended in EPRI Report 1021467]...peer review resulted in 26 findings and observations," to which the licensee stated that "all 26 findings have been addressed in the SSPSS 2011 model update."
 - a. The safety evaluation (SE) makes changes to the language for the EPRI Report. Because of this, EPRI Report will be updated to "-A" version within 3 months of receipt of the final SE. Please confirm that the licensee's PRA conforms to the EPRI Report, as modified by the NRC staff's SE on January 18, 2012 (ADAMS Accession No. ML11325A340), or identify any areas where the quality of the PRA differs from that described in the version that will be developed after EPRI incorporates the SE into the Report.

Enclosure

b. The licensee states:

"All 26 internal flooding peer review findings have been addressed in the SSPSS-2011 model update. The three "B" level findings and associated disposition are summarized in the table below. Other findings had either a minor impact on the model or were related to improving documentation."

The NRC safety evaluation for EPRI Technical Report (TR) 1021467 "Nondestructive Evaluation: Probabilistic Risk Assessment Technical Adequacy Guidance for Risk-Informed In-Service Inspection Programs" in Attachment 1, Table 1 clarifies that for a supporting requirement to be considered met, all relevant peer and other independent review findings shall have been addressed and, as necessary, applicable changes made to PRA models, methods, and documentation. Please provide the NRC staff confirmation that all findings for those supporting requirements, as identified in EPRI TR 1021467, are properly dispositioned and now meet the Capability Category appropriate for that supporting requirement (SR) as reflected in EPRI TR 1021467, as approved by the NRC SE. If any SR is not met at the appropriate Capability Category level, please describe the finding and its disposition for this application.

P. Freeman

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If you have questions, you can contact me at 301-415-3100 and/or John.Lamb@nrc.gov.

Sincerely,

/ra/

John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosure:
Request for Additional Information

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RidsRgn1MailCenter Resource

RidsNrrLAABaxter Resource

ADAMS ACCESSION NO.: ML120390570

***via email**

OFFICE	LPL1-2/PM	LPL1-2/LA	APLA/BC	LPL1-2/BC
NAME	JLamb	ABaxter	DHarrison*	MKhanna
DATE	2/14/2012	2/13/2012	2/7/2012	2/15/2012

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