

DATE ADDED  
February 21, 2012

Mr. Michael J. Pacilio  
President and Chief Nuclear Officer  
Exelon Nuclear  
4300 Winfield Road  
Warrenville, IL 60555

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION: REQUEST FOR  
ADDITIONAL INFORMATION RELATED TO THE RESPONSE TO LICENSE  
RENEWAL COMMITMENT NO. 10 REGARDING THERMAL AGING  
IRRADIATION EMBRITTLEMENT OF CAST AUSTENITIC STAINLESS STEEL  
(TAC NO. ME7123)

Dear Mr. Pacilio:

By letter dated April 9, 2008, AmerGen Energy Company, the licensee, submitted for staff review and approval a response to Commitment No. 10 stated in NUREG-1875, Volume 2, "Safety Evaluation Report Related to the License Renewal of Oyster Creek Generating Station [(OCNGS)]." This commitment concerns thermal aging and neutron irradiation embrittlement of OCNGS reactor vessel internals components made of cast austenitic stainless steel materials. The staff is reviewing the information contained in the license renewal commitment response and has identified, in the enclosure, an area where additional information is needed to complete the review.

The request for additional information (RAI) was discussed with David Helker of your staff, and a mutually agreeable date for the response is within 45 days from the date of this letter. If you have any questions, please contact me by telephone at (301) 415-4054 or by e-mail at [Heather.Jones@nrc.gov](mailto:Heather.Jones@nrc.gov).

Sincerely,

/RA/

Heather M. Jones, Project Manager  
Program Operations Branch  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosure:  
As stated

cc w/encl: Listserv

REQUEST FOR ADDITIONAL INFORMATION  
BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
COMMITMENT NO. 10 OF NUREG-1875, VOLUME 1, "SAFETY EVALUATION REPORT  
RELATED TO THE LICENSE RENEWAL OF  
OYSTER CREEK NUCLEAR GENERATING STATION"  
AMERGEN ENERGY COMPANY  
DOCKET NO. 50-219

RAI-1

Regarding the fuel support piece, it was stated that, "[t]he loading on the fuel support piece is mostly in compression and as such any loss of fracture toughness is not expected to affect the structural integrity of the component." Please elaborate on the locations of the fuel support piece which are under tensile stresses, provide the maximum tensile stress value and confirm whether the highly stressed areas are accessible in the proposed supplemental inspections. If all accessible surfaces of the fuel support piece are under compressive stresses, what information do you expect to obtain from the proposed supplemental inspections? Further, discuss the inspection results documented to date.

Principle contributor: Simon Sheng, NRR

ENCLOSURE

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**/RA/**  
Heather M. Jones, Project Manager  
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Office of Nuclear Reactor Regulation

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NAME	IKing	HJones	YDiaz-Sanabria	HJones
DATE	02/8/2012	02/09/2012	02/21/2012	02/21/2012

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Letter to M. Pacilio from H. Jones dated February 21, 2012

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION: REQUEST FOR  
ADDITIONAL INFORMATION RELATED TO THE RESPONSE TO LICENSE  
RENEWAL COMMITMENT NO. 10 REGARDING THERMAL AGING  
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(TAC NO. ME7123)

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