

February 3, 2012

Neil Wilmschurst  
Vice President and Chief Nuclear Officer  
Electric Power Research Institute  
1300 West W. T. Harris Boulevard  
Charlotte, North Carolina 28262-8550

SUBJECT: ENDORSEMENT TO THE FINAL SAFETY EVALUATION OF ELECTRIC  
POWER RESEARCH INSTITUTE (EPRI) REPORT 1016596,  
“MATERIALS RELIABILITY PROGRAM (MRP): PRESSURIZED WATER  
REACTOR INTERNALS INSPECTION AND EVALUATION GUIDELINES  
(MRP-227-Rev.0)”

Dear Mr. Wilmschurst:

By letter dated January 12, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML090160204), the EPRI submitted for U.S. Nuclear Regulatory Commission (NRC) staff review and approval EPRI Report 1016596, “Materials Reliability Program (MRP): Pressurized Water Reactor Internals Inspection and Evaluation Guidelines (MRP-227-Rev.0).” On June 22, 2011, the NRC issued Revision 0 of the Safety Evaluation (SE) for MRP-227 (ADAMS Accession No. ML111600498) and on December 16, 2011, issued Revision 1 of the SE (ADAMS Accession No. ML11308A770). On January 9, 2012, the EPRI submitted to the NRC the final MRP-227-A. The seven parts as received from the EPRI are available in ADAMS as Accession Nos. ML12017A193 – ML12017A199.

Topical Report (TR) MRP-227-A, contains an updated discussion of the technical basis for the development of an aging management program (AMP) for reactor vessel internal components in the PWR vessels supplied by Westinghouse Electric Company, Babcock and Wilcox, and Combustion Engineering. TR MRP-227-A, provides inspection and evaluation guidelines as part of an AMP for use by the licensees.

The NRC staff has found that TR MRP-227-A is acceptable for referencing in licensing applications for PWR internals inspection and evaluation and endorses the TR for that purpose. The staff’s final evaluation of the MRP-227, Revision 0 report, included eight plant-specific action items that will need to be considered by licensees.

Our endorsement applies only to material provided in TR MRP-227-A. We do not intend to repeat our review of the acceptable material described in the TR. When the TR appears as a reference in license amendment requests or license renewal (LR) applications, our review will ensure that the material presented applies to the specific plant involved. License amendment requests or references to this TR in LR applications that deviate from this TR will be subject to a plant-specific review in accordance with applicable review standards.

The NRC considers the endorsement of this TR as a very positive step in dealing with aging management. Our staff appreciates the responses to requests for additional information and the updates to the TR as we worked through the process. If you have any questions about the TR or the TR process, please contact Sheldon Stuchell at [Sheldon.stuchell@nrc.gov](mailto:Sheldon.stuchell@nrc.gov) or 301-415-1847.

Sincerely,

**/RA/**

Robert A. Nelson, Deputy Director  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

Project No. 669

N. Wilmshurst

-2-

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Sincerely,

**/RA/**

Robert A. Nelson, Deputy Director  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

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