



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 9, 2012

LICENSEE: FirstEnergy Nuclear Operating Company

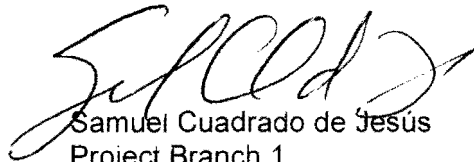
FACILITY: Davis-Besse Nuclear Power Station

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JUNE 16, 2011,
BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND
FIRSTENERGY NUCLEAR OPERATING COMPANY, CONCERNING
REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE
DAVIS-BESSE NUCLEAR POWER STATION LICENSE RENEWAL
APPLICATION (TAC. NO. ME4640)

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on June 16, 2011, to discuss and clarify the staff's concerns related to the Davis-Besse Nuclear Power Station license renewal application.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a description of the staff concerns discussed with the applicant. A brief description on the status of the items is also included.

The applicant had an opportunity to comment on this summary.


Samuel Cuadrado de Jesús
Project Branch 1
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures
As stated

cc w/encls: Listserv

SUMMARY OF TELEPHONE CONFERENCE CALL
DAVIS-BESSE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS
June 16, 2011

PARTICIPANTS

Samuel Cuadrado de Jesús
James Medoff
Steve Dort
Kathy Nesser
John Hartigan
Larry Hinkle

AFFILIATIONS

U.S. Nuclear Regulatory Commission (NRC)
NRC
FirstEnergy Nuclear Operating Company (FENOC)
FENOC
FENOC
FENOC

SUMMARY OF TELEPHONE CONFERENCE CALL
DAVIS-BESSE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION
June 16, 2011

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on June 16, 2011, to discuss and clarify the following concerns related to the Davis-Besse Nuclear Power Station (Davis-Besse) license renewal application (LRA).

Discuss absence of a time-limited aging analysis (TLAA) on their bases for non-Class 1, non-piping components

Discussion:

The staff requested the applicant to explain the absence of a TLAA on their bases for non-Class 1, non-piping components.

The applicant responded by explaining that non-Class 1 components reanalyzed as Class 1 components are addressed under the Class 1 Section of the LRA.

The staff understood the applicant's explanation and stated that no request for additional information (RAI) is necessary.

RAI 4.1-2

Discussion:

The staff requested the applicant to discuss extension of the scope of RAI 4.1-2 to both the time-dependent J-integral analysis and the time-dependent fatigue flaw growth analysis in Structural Integrity Associates (SIA) Topical Report No. SIR-99-040, Revision 1, "ASME Code Case N-481 of Davis-Besse Reactor Coolant Pumps" (ADAMS Accession No. ML011200090). The staff stated that the scope of the RAI, as issued, only talked about the fatigue flaw growth analysis in the report, which forms the basis for applying VT-1 or EVT-1 visual examinations of the outside surfaces of the reactor coolant pump (RCP) casing welds in lieu of the ultrasonic test (UT) examinations that would be required by the ASME Section XI Code of Record. The staff also stated that there is a possibility, that 10 CFR 50.55a may require the applicant to update to a more recent edition of the ASME Section XI Code that has incorporated the visual examinations requirements for the pump casings. If this is the case, the staff stated that it will need to know whether the applicant's current licensing basis (CLB) is still relying on the SIA report to support using the visual methods cited in the updated ASME Section XI Code of Record.

The applicant responded by stating that the response to RAI 4.1-2 as presently drafted only addresses time-dependent fatigue flaw growth analysis. The applicant stated that the response is included in the letter (L-11-203) that is to be sent to the staff on Friday, June 17, 2011. Because the response was already drafted and already went through the applicant's submittal process, a supplement to the response will be submitted at a later date (most likely with the letter due June 24, 2011) to address the time-dependent J-integral analysis.

In addition, the applicant also stated that the applicable ASME Code for the current (third) 10-year inspection interval for Davis-Besse is ASME Section XI, 1995 Edition, through the 1996 Addenda and that the interval does not end until September 20, 2012. Therefore, the Davis-Besse CLB still relies on the SIA Topical Report SIR-99-040.

Action: NRC project manager and applicant's license renewal project manager will discuss submittal date via telephone.

Topic updated safety analysis report (USAR) Appendix 5A design basis for reactor coolant pump (RCP) flywheel integrity

Discussion:

The staff requested the applicant to discuss their USAR Appendix 5A design basis for RCP flywheel integrity. The staff agrees that the reference temperature nil ductility (RT_{ndt}) analysis for the flywheels in that appendix does not need to include a time-dependent neutron fluence-based ΔRT_{ndt} adjustment in the manner that they are included in the RT_{ndt} analyses for the reactor vessel beltline components (i.e., beltline base metals and weld components). However, USAR Appendix 5A states that the Standard Review Plan (SRP) Section 5.4.1.1 acceptance basis is an 80°F difference basis between the RT_{ndt} value and the operating temperature (which according to the USAR Appendix, puts the minimum operating temperature at 120°F). Contrary to this statement, the staff determined that SRP 5.4.1.1 states (recommends) that the difference between the RT_{ndt} value and the operating temperature should be at least 100°F, which for full conformance with the SRP basis would dictate a minimum operating temperature of 140°F for the RCP flywheels at Davis-Besse. The USAR Section 5A also states that Section 3.1 of the USAR gives the 120°F operating temperature basis for the flywheels, but the staff could not find any such basis in USAR Section 3.1. The staff further stated that SRP Section 5.4.1.1 on flywheel integrity (which the applicant uses as the USAR Appendix 5A basis) states that the RT_{ndt} values for the RCP flywheel plate will be based on actual drop-weight testing results; however, the applicant established the RT_{ndt} value for the SA-533 flywheel plates materials using generic application of the RT_{ndt} values for their SA-533 plate materials for the reactor vessel (RV) beltline materials (pick 40°F as the highest value). The staff stated that this is a CLB issue.

The applicant responded by stating that its intention was not to completely comply with SRP 5.4.1.1.

The staff suggested that the applicant look into this, not as a license renewal issue, but rather as a CLB issue. The staff stated that no RAI will be issued.

Reactor Vessel Internal (RVI) components

Discussion:

The staff stated that the discrepancy between the applicant's Technical Specification (TS) 5.5.4 inspection requirements for its RVI vent valve discs and its plant-specific Pressurized-Water Reactor (PWR) Vessel Internals Program criteria that needs to be discussed. The staff also stated that the adequacy of the aging management reviews (AMRs) for the RVI components will need to be discussed as well.

The applicant stated that it does not believe that a discrepancy exists between the TS 5.5.4 inspection requirements for its RVI vent valve discs and its plant-specific PWR Vessel Internals Program criteria that is based on MRP-227. The applicant also stated that the TS requires the disc inspection each refueling outage versus MRP-227 that requires the inspection each 10-year interval. Therefore, MRP-227 inspection frequency is satisfied. However, this may require further discussion, as the staff lead reviewer, Ganesh Cheruvenki, was not in attendance.

The AMR of the RVI was briefly discussed. The applicant suggested that an LRA supplement could be submitted that would revise the AMR for RVI and that it would be based on MRP-227 along with the guidance provided in NUREG-1801, Revision 2 (GALL Report). The staff stated that the concerns related to the RVI would be discussed internally (to include Ganesh Cheruvenki) and a response provided to the applicant at a later date.

There was no further discussion, and the call was concluded.

Memorandum to FirstEnergy Nuclear Operating Company from S. CuadradoDeJesus dated February 9, 2012

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/RA/

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| NAME | YEdmonds/IKing for | SCuadrado de Jesús | DMorey |
| DATE | 2/ 9 /12 | 2/ 09 /12 | 2/ 09/12 |

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