



Monticello Nuclear Generating Plant  
2807 W County Road 75  
Monticello, MN 55362

January 17, 2012

L-MT-12-006  
10 CFR 50.73

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555

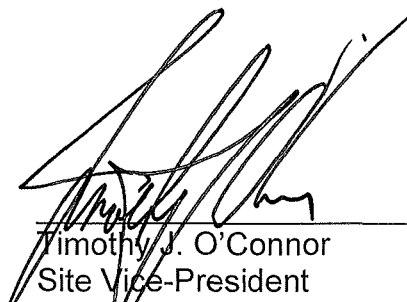
Monticello Nuclear Generating Plant  
Docket 50-263  
Renewed Facility Operating License No. DPR-22

LER 2011-009 "Automatic Reactor Scram While Performing Turbine – Generator Testing"

A Licensee Event Report (LER) for this occurrence is attached.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.



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Timothy J. O'Connor  
Site Vice-President  
Monticello Nuclear Generating Plant  
Northern States Power Company-Minnesota

Enclosure

cc: Regional Administrator, Region III, USNRC  
Project Manager, Monticello Nuclear Generating Plant, USNRC  
Resident Inspector, Monticello Nuclear Generating Plant, USNRC

<b>NRC FORM 366</b> <b>U.S. NUCLEAR REGULATORY COMMISSION</b> (10-2010)				<b>APPROVED BY OMB NO. 3150-0104</b> <b>EXPIRES 10/31/2013</b>  Estimated burden per response to comply with this mandatory information collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.																																											
<b>LICENSEE EVENT REPORT (LER)</b> (See reverse for required number of digits/characters for each block)																																															
<b>1. FACILITY NAME</b> Monticello Nuclear Generating Plant				<b>2. DOCKET NUMBER</b> 05000 - 263		<b>3. PAGE</b> 1 OF 3																																									
<b>4. TITLE</b> Automatic Reactor Scram While Performing Turbine – Generator Testing																																															
<b>5. EVENT DATE</b>			<b>6. LER NUMBER</b>			<b>7. REPORT DATE</b>			<b>8. OTHER FACILITIES INVOLVED</b>																																						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER																																					
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<b>9. OPERATING MODE</b>  <div style="text-align: center; font-size: 24pt;">1</div>			<b>11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:</b> (Check all that apply) <table style="width: 100%; border: none;"> <tr> <td><input type="checkbox"/> 20.2201(b)</td> <td><input type="checkbox"/> 20.2203(a)(3)(i)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(C)</td> <td><input type="checkbox"/> 50.73(a)(2)(vii)</td> </tr> <tr> <td><input type="checkbox"/> 20.2201(d)</td> <td><input type="checkbox"/> 20.2203(a)(3)(ii)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(A)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(1)</td> <td><input type="checkbox"/> 20.2203(a)(4)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)(B)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(B)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(i)</td> <td><input type="checkbox"/> 50.36(c)(1)(i)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(iii)</td> <td><input type="checkbox"/> 50.73(a)(2)(ix)(A)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(ii)</td> <td><input type="checkbox"/> 50.36(c)(1)(ii)(A)</td> <td><input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(x)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(iii)</td> <td><input type="checkbox"/> 50.36(c)(2)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(A)</td> <td><input type="checkbox"/> 73.71(a)(4)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(iv)</td> <td><input type="checkbox"/> 50.46(a)(3)(ii)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(B)</td> <td><input type="checkbox"/> 73.71(a)(5)</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(v)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(A)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(C)</td> <td><input type="checkbox"/> OTHER</td> </tr> <tr> <td><input type="checkbox"/> 20.2203(a)(2)(vi)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)(B)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)(D)</td> <td>Specify in Abstract below or in NRC Form 366A</td> </tr> </table>									<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A
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<b>10. POWER LEVEL</b>  <div style="text-align: center; font-size: 24pt;">90%</div>																																															
<b>12. LICENSEE CONTACT FOR THIS LER</b>																																															
<b>NAME</b> Carrie Fosaaen								<b>TELEPHONE NUMBER (Include Area Code)</b> 763-295-1357																																							
<b>13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT</b>																																															
CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX																																						
<b>14. SUPPLEMENTAL REPORT EXPECTED</b>						<b>15. EXPECTED SUBMISSION DATE</b>		MONTH	DAY	YEAR																																					
<input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input type="checkbox"/> NO								02	29	2012																																					
<b>ABSTRACT</b> (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)																																															
On November 19 <sup>th</sup> , 2011, at approximately 2312 CST, during performance of regularly scheduled Turbine-Generator Quarterly Surveillance, an unplanned reactor scram occurred. Following the reactor scram, reactor water level lowered below the Group II isolation initiation setpoint (+9 in) and an actuation of Primary Containment Isolation System occurred.																																															
The direct cause of the scram was the actuation of the Main Turbine acceleration relay (load rejection) pressure switches. The root cause is under investigation by the site. A supplement to this Licensee Event Report will be submitted following completion of the investigation and will outline corrective actions to address the root cause.																																															

NRC FORM 366A (10-2010)		<b>LICENSEE EVENT REPORT (LER) CONTINUATION SHEET</b>		U.S. NUCLEAR REGULATORY COMMISSION	
1. FACILITY NAME		2. DOCKET	6. LER NUMBER		3. PAGE
Monticello Nuclear Generating Plant		05000 -263	YEAR	SEQUENTIAL NUMBER	REV NO.
			2011	- 009	- 00
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**NARRATIVE**

Energy industry identification system (EIS) codes are identified in the text within brackets [xx].

**EVENT DESCRIPTION**

Prior to the event, Monticello Nuclear Generating Plant was in Mode 1 at approximately 90% power.

On November 19, 2011, at approximately 2312 CST, the plant scrammed while performing a Turbine – Generator Quarterly Surveillance Test, which tests the operation of the Speed/Load Changer and Turbine Bypass Valves [V]. The Speed/Load Changer was being lowered to close the Control Valves [V] and concurrently open the Bypass Valves when a Reactor half scram was received followed by a full Reactor scram due to both channels of the Turbine-Generator load reject trip relays [RLY] which receive their signal from oil pressure sensing switches [PIS]. Following the reactor scram, reactor water level lowered below the Group II isolation initiation setpoint (+9 in) and an actuation of Primary Containment Isolation System (PCIS) occurred.

Control Rods fully inserted as expected in response to the Reactor Protection System [JC] (RPS) actuation. Post scram, Reactor Vessel [RPV] water level was controlled using the Feedwater [SJ] and Condensate [SD] systems. No other safety systems actuated or were required to actuate. There was no inoperable equipment at the start of the event that contributed to the event. Off-site power was available and both Emergency Diesel Generators [DG] were operable and available. Crew recognition, response and decision making enabled effective management of the transient.

**EVENT ANALYSIS**

This event is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A) – System Actuation. This event is not considered a safety system functional failure.

**CAUSE**

The direct cause of the scram was the actuation of the Main Turbine acceleration relay (load rejection) pressure switches. The root cause investigation is in progress. A supplement to the Licensee Event Report will be submitted upon completion of the investigation.

**SAFETY SIGNIFICANCE**

The safety objective of both RPS and PCIS are to provide timely protection at the onset of conditions that could challenge the integrity of the fuel barrier and nuclear system process barriers. The RPS prevents the release of radioactive material from the fuel and nuclear system process barriers by terminating excessive temperature and pressure increases through the initiation of an automatic plant shutdown. PCIS prevents release of radioactive materials by isolating the reactor vessel and closing containment where required. For this event, the RPS, PCIS, and plant safety systems functioned as designed and fuel and nuclear system process barriers remained intact. Consequently, the event did not have an adverse impact on the health and safety of the public.

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Monticello Nuclear Generating Plant	05000 -263	YEAR	SEQUENTIAL NUMBER	REV NO.	3 OF 3
		2011	- 009	- 00	
<b>NARRATIVE</b> <b>CORRECTIVE ACTIONS</b> <p>The root cause investigation is in progress. A supplement to the Licensee Event Report will be submitted upon completion of the investigation and will outline corrective actions to address the root cause.</p> <b>PREVIOUS SIMILAR EVENTS</b> <p>There have been no similar licensee event reports in the past three years.</p>					