

FEB 28 1976

Docket No. 50-263

Northern States Power Company  
ATTN: Mr. L. O. Mayer, Director  
Nuclear Support Services  
414 Nicollet Mall  
Minneapolis, Minnesota 55401

Gentlemen:

RE: MONTICELLO NUCLEAR GENERATING PLANT      LICENSE NO.: DPR-22

On May 5, 1975, the Commission published Appendix I to 10 CFR Part 50 which sets forth numerical guides for design objectives and limiting conditions for operation to meet the criterion "as low as practicable" regarding releases of radioactive materials in effluents from light-water-cooled nuclear reactors. Section V.B of Appendix I requires the holders of permits or licenses authorizing the operation of light-water-cooled reactors, for which application was filed prior to January 2, 1971, to file with the Commission by June 4, 1976:

1. Such information as is necessary to evaluate the means employed for keeping levels of radioactivity in effluents to unrestricted areas as low as practicable, including all such information as is required by paragraphs 50.34a(b) and (c) not already contained in his application; and
2. Plans and proposed Technical Specifications developed for the purpose of keeping releases of radioactive materials in unrestricted areas during normal reactor operations, including expected operational occurrences, as low as practicable.

Guidance for filing the information required by Section V.B of Appendix I is set forth in Enclosure 1 to this letter.

Enclosure 1 makes reference to proposed Regulatory Guides 1.AA, thru 1.FF. These guides have been published in draft form and are available in the Commission's public document room. Copies of these guides are being forwarded to you under separate cover. We are preparing additional guidance regarding the formulation of Technical Specifications to implement Appendix I which will be forwarded to you in the near future. It is recommended that proposals to modify Technical Specifications related to Appendix I be deferred pending the completion of this guidance in order to obtain a consistent format for radiological effluent release Technical Specifications.

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Enclosure 2 provides a list of information we will need to evaluate your facility.

The staff is in the process of determining whether plants for which application was filed prior to January 2, 1971, can be treated in a manner similar to the treatment given to later plants in the Commission's September 4, 1975, Amendment to Appendix I. On this basis, the information called for by Enclosures 1 and 2 would be sufficient to determine compliance with the criteria set forth in Section II, paragraphs A, B, and C of Appendix I and the design objectives set forth in the Concluding Statement of Position of the Regulatory Staff in Docket RM-50-2, which is reproduced as an Annex to Appendix I as amended September 4, 1975. If further information is required to satisfy a plant specific assessment under Section II.D of Appendix I, we will so inform you at a later time.

Within 30 days of the receipt of this letter, you should inform us of the measures you intend to adopt or propose to meet the requirements of paragraph 50.34a. We suggest that a meeting between your staff and the NRC staff be arranged shortly thereafter to discuss the submittal of data and methods of evaluation.

Sincerely,

Original Signed by:  
Dennis L. Ziemann.

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Enclosures:

- Guidance to Holders of Permits to Construct or Licenses to Operate Light-Water-Cooled Reactors for which Application was Filed Prior to 1/2/71 to Meet the Requirements of Appendix I to 10 CFR Part 50
- Additional Information Needed from Holders of Permits or Licenses to Construct or Operate Light-Water-Cooled Reactors for Which Application was Filed Prior to 1/2/71

cc: with enclosures  
See next page

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cc w/enclosures:

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RE: MONTICELLO NUCLEAR GENERATING PLANT

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As you undoubtedly know, the current shortage of offsite fuel storage space and fuel reprocessing capability has curtailed the ability of some operating reactor facilities to ship spent fuel to an offsite location. This situation will probably continue to exist for several more years. Consequently, spent fuel is accumulating in onsite spent fuel storage pools; and as the available onsite storage space is used up, more facilities will not have adequate capacity to discharge a full reactor core to the storage pool.

The capability to discharge a full core from the reactor vessel to the storage pool is considered to be an operational consideration rather than a safety problem. However, it should be noted that in some facilities, complete core unloading may be necessary to perform some of the required reactor coolant system inservice inspections. Thus, the inability to discharge a full core could preclude compliance with these inspection requirements and continued operation.

We suggest that you review the inservice inspection requirements in the NRC regulations and your Technical Specifications to determine whether they require discharging fuel, and review your schedule for performing these inspections in view of your projected fuel storage capability. If you find that your current inservice inspection schedules cannot be accommodated within your projected ability to discharge fuel, you should consider advancing your inspection schedules accordingly or taking other action to alleviate the problem.

Sincerely,

Original Signed by:  
Dennis L. Ziemann

Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

cc: See next page

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