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SUBJECT: Provides supplemental response to loss of all alternating current power info, per 10CFR50.63(c)(1).

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TITLE: OR Submittal: Station Blackout (USI A-44) 10CFR50.63, MPA A-22

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March 29, 1990

10 CFR Part 50
Section 50.63(c)(1)

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Supplemental Response to Loss of All Alternating
Current Power Information Required by
10 CFR Part 50, Section 50.63(c)(1)

- Reference: 1) Letter from Mr. A C Thadani, (NRC) to Mr. A Marion (NUMARC), dated January 3, 1990, "Supplemental Guidance Documents for Addressing the Station Blackout Issue".
- 2) Letter from T M Parker (NSP) to the Director of NRR (NRC), dated October 17, 1989, "Loss of All Alternating Current Power Information Required by 10 CFR Part 50, Section 50.63(c)(1)".

The following supplemental response is being submitted to provide the information requested by Reference 1. The information forwarded in Reference 2 was based on NUMARC 87-00 guidance and complies with subsequent clarification provided in Reference 1.

The following deviations from the accepted NUMARC 87-00 guidance were not identified as part of the information forwarded by Reference 2:

- 1) A computer based analytical tool was used, instead of the NUMARC methodology, to calculate the room heatup for the Drywell and Control Room under Station Blackout conditions. This analytical tool provides a more accurate and realistic prediction of what the room temperature would be during a station blackout.
- 2) A heatup calculation of the torus room was not performed because the geometry of the torus room did not lend itself to the NUMARC methodology. Instead it was conservatively assumed that the temperature of the torus room is the temperature of the hottest heat source in that room. The hottest heat source in the torus room is the High Pressure Coolant Injection/Reactor Core Isolation Cooling System steam piping with a temperature of 160°F on the surface of the insulation.

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Dir of NRR
March 29, 1990
Page 2

Section C.3 of the information forwarded by Reference 2 indicated a need to install a nitrogen supply to the air operated valves which drain condensate from the steam supply line to the High Pressure Coolant Injection system turbine and the steam exhaust line from the High Pressure Coolant Injection system turbine. Additional analysis has been performed which indicated that this modification is no longer necessary.

A correction to C.4.a of the information forwarded by Reference 2 has also been identified. The note indicated that there was only one component located in the Steam Tunnel which was required to operate during a Station Blackout. However, there are two. Both components are DC powered motor operated valves which are required to operate only at the initiation of the Station Blackout. Because they are not required to operate during an extended Station Blackout they do not need to be capable of performing at elevated temperatures resulting from the loss of ventilation. It is not necessary to demonstrate reasonable assurance of operability for the duration of the Station Blackout.

With regard to the diesel generator target reliability chosen for Monticello of 0.95, it is understood that this value is to be maintained.

Please contact us if you have any questions related to the information we have provided.



Thomas M Parker
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Nuclear Support Services

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